

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE July 15, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE 717-948-8551

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: JUNE ,1984.
3. LICENSED THERMAL POWER (MWT): 2535.
4. NAMEPLATE RATING (GROSS MWE): 871.
5. DESIGN ELECTRICAL RATING (NET MWE): 819.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

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9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)

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10. REASONS FOR RESTRICTIONS, IF ANY:

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	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	720.	4367.	86160.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31751.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	36.2
20. UNIT AVAILABILITY FACTOR	0.0	0.0	36.2
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	35.4
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	33.8
23. UNIT FORCED OUTAGE RATE	100.0	100.0	60.5

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)

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25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

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IE 24  
 1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE July 15, 1984  
COMPLETED BY C. W. Smyth  
TELEPHONE 717-948-8551

MONTH: JUNE

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.
2	0.
3	0.
4	0.
5	0.
6	0.
7	0.
8	0.
9	0.
10	0.
11	0.
12	0.
13	0.
14	0.
15	0.
16	0.

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
17	0.
18	0.
19	0.
20	0.
21	0.
22	0.
23	0.
24	0.
25	0.
26	0.
27	0.
28	0.
29	0.
30	0.
31	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289  
 UNIT NAME TMI-I  
 DATE July 15, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE 717-948-8551

REPORT MONTH JUNE, 1984

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	84-01-07	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

OPERATIONS SUMMARY

The unit was in cold shutdown the entire report period by order of the NRC. Core cooling was provided by the Decay Heat Removal System (except during short periods of high grid load). On June 4, partial draining of the RCS was commenced to enable valve repacking and other maintenance work that was identified during Hot Functional Testing.

On June 18 the RCS was filled, vented and was pressurized on June 19 to enable running of a RCP. As a result of slight increases in boron and tritium concentrations in the "B" OTSG secondary while the RCS was pressurized to 300 PSI, the RCS was again partially drained to enable bubble testing of "B" OTSG tubes. This testing was performed on June 25. RCS work the remainder of the month involved entry into both OTSGs for eddy current and tube plug verification.

MAJOR SAFETY RELATED MAINTENANCE

During the month the following major maintenance items were completed.

Reactor Coolant Pump RC-P-1B

Fabricated, installed, and painted new Nuclear Service Closed Cooling Water Piping to RC-P-1B oil and air coolers. The old piping was damaged and/or lost during RC-P-1B overhaul. All work is now complete.

Makeup Valve MU-V-205

Disassembled valve and replaced the stem and disc assembly because the tip of the disc was broken off. The problem was identified by low flow through valve. All work was complete and tested with satisfactory results.

Chemical Addition Sample Valve CA-V-4B

Replaced stem and disc assembly because old stem was scored/scratched resulting in excessive packing leak off. Valve was local leak rate tested after repairs and all work was complete.

Once Through Steam Generators RC-H-1A/B

Removed the upper and lower manways and handholes on "B" OTSG and upper manway and handhole on "A" OTSG for bubble/drip testing because of tube leakage identified by increased boron in the secondary side. Eddy current inspections commenced and will continue in July followed by repairs to identified leaking tubes.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1988 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



GPU Nuclear Corporation  
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TELEX 84-2386  
Writer's Direct Dial Number:

July 13, 1984  
5211-84-2167

Office of Management Information and Program Control  
Attn: W. C. McDonald  
c/o Distribution Services Branch DPC, ADM  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. McDonald:

Three Mile Island Nuclear Station, Unit I (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
June Monthly Operating Report

Enclosed please find two (2) copies of the June Operating Report for Three Mile Island Nuclear Station Unit.

Sincerely,

  
H. D. Hukill,  
Director, TMI-1

HDH:mle  
Attachments

cc: V. Stello  
Dr. E. T. Murley

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