NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Quad-Cities Nuclear Power Station, Unit 1	0 5 0 0 0 2 5 4	1 OF 0 2

TITLE (4)

Reactor Scram on Spurious Main Steam Line High Flow Signal

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 8	2 8	8 4	8 4	0 1 6	0 0	0 9	2 6	8 4	NA	0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	0 6 0	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)	73.71(b)	
		20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)	
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. Eagle	3 0 9 6 5 4 - 2 2 4 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 28, 1984, Unit One was in the RUN mode at approximately 60 percent core thermal power. At 1352 hours, a high Main Steam Line Flow signal was received in the Control Room. This resulted in a Group I Isolation and an automatic Reactor scram. The cause of the scram was that Instrument Maintenance personnel, while conducting a routine surveillance test, inadvertently created a low pressure transient in one division of the Main Steam High Flow Trip Logic System causing a spurious isolation signal. Proper operation of all Reactor safety systems and prompt Operator action minimized the safety implications of this occurrence.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Quad-Cities Nuclear Power Station, Unit 1	0 5 0 0 0 2 5 4	8 4	— 0 1 6	— 0 0	0 2 OF 0 2

TEXT *(If more space is required, use additional NRC Form 366A's)* (17)

Event Description

On August 28, 1984, at 1352 hours, Unit One was in the RUN mode at approximately 60 percent core thermal power. While Instrument Maintenance personnel were performing the Main Steam Line (MSL) Hi Flow Surveillance, QIS 21-1, a Group I Isolation was received and an automatic Reactor scram occurred. The 901-5 Panel in the Control Room indicated that the first trip signal was caused by high main steam line flow. All control rods inserted to 00 position and a normal trip recovery was initiated. All Reactor safety systems were operable and functioned as designed, therefore, the safety implications of this occurrence were minimal.

This event is being reported as required by the Code of Federal Regulations, 10 CFR 50.73(a)(2)(iv).

Cause

The cause of this event is personnel error. Sixteen differential pressure (dP) switches, four per MSL, constitute the MSL High Flow Trip Logic. The switches are wired in a one-out-of-two-twice logic arrangement to give a Group I Isolation on high steam line flow. While calibrating one of the switches, the Instrument Mechanic inadvertently opened its isolation valve on the low pressure side prior to pressurizing the instrument to approximate Reactor pressure. This resulted in a low pressure transient which satisfied the one-out-of-two-twice logic for the corresponding MSL. Therefore, a Group I Isolation and subsequent Reactor scram occurred.

Corrective Action

The immediate corrective action was to ensure all control rods were fully inserted and initiate a normal scram recovery. The Instrument Mechanic involved was carefully counseled on his responsibilities by Station's Management. No further corrective action is deemed necessary at this time due to the fact that this Instrument Mechanic has, over the years, successfully demonstrated his proficiency at this exact task hundreds of times without error.

Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

NJK-84-293

September 26, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One

Enclosed please find Licensee Event Report number 84-016 for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)-(2)(iv), which requires reporting of any event or condition that resulted in manual or automatic actuation of any engineered safety feature.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:HQD/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center
NRC Region III

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