



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NOS. 100 AND 102 TO FACILITY OPERATING LICENSE

NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1. INTRODUCTION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements" which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications are required. A number of items which require Technical Specifications were scheduled for implementation by December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-02. Generic Letter 83-02 was issued to all Boiling Water Reactor (BWR) licensees on January 10, 1983. In Generic Letter 83-02 the staff requested licensees to:

- a. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the generic letter, and
- b. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated February 11, 1982, Philadelphia Electric Company (licensee) submitted a Technical Specification change request for Peach Bottom Units 2 and 3 which included many items covered by Generic Letter 83-02. By letter dated August 24, 1983, the licensee responded to Generic Letter 83-02 and submitted a Technical Specification change request for Peach Bottom Units 2 and 3. The later submittals revised some of the Technical Specifications proposed in the February 11, 1982 letter. This evaluation covers the following TMI Action Plan items:

- a. Reporting of Safety and Relief Valve Failures and Challenges (II.K.3.3)
- b. RCIC Restart (II.K.3.13)
- c. Isolation of HPCI and RCIC Modification (II.K.3.15)

2. DISCUSSION AND EVALUATION

A. Reporting of Safety/Relief Valve Failures and Challenges (II.K.3.3)

In Generic Letter 83-02, the staff requested licensees to formalize the reporting requirements for safety/relief valve failures and challenges. The licensee has proposed Technical Specifications which will require the licensee to report the failures promptly with written follow-up, and the challenges in an annual report. The proposed Technical Specifications meet the intent of the guidance provided in Generic Letter 83-02. Therefore we find it acceptable.

B. RCIC Restart (II.K.3.13)

TMI Action Plan Item II.K.3.13 recommends modifications to the Reactor Core Isolation Cooling System (RCIC) such that:

1. The system will restart on a subsequent low water level signal after it has been terminated by a high water level in the reactor vessel.

In Generic Letter 83-02, the staff provided guidance on necessary changes in the Technical Specifications for implementation of the modifications. The proposed changes in Technical Specifications for RCIC are in response to Generic Letter 83-02. We have reviewed the proposed changes in the Technical Specifications and determined that the changes are consistent with the guidance provided in Generic Letter 83-02. We find the changes acceptable.

C. Isolation of HPCI and RCIC Modifications (II.K.3.15)

TMI Action Plan Item II.K.3.15 recommends that the pipe-break-detection circuitry should be modified so that pressure spikes resulting from High-Pressure Coolant Injection (HPCI) and RCIC system initiation will not cause inadvertent system isolation. The licensee has completed the modification recommended by this item.

In Generic Letter 83-02, the staff provided guidance on the scope of the Technical Specifications required by this item. The licensee has proposed changes in the Technical Specifications for Peach Bottom Units 2 and 3. We have reviewed the proposed changes for both Units 2 and 3 and determined that the Technical Specifications cover the surveillance requirements on the time delay relay included in HPCI and RCIC systems. The proposed changes are consistent with our guidance in Generic Letter 83-02. Therefore, we find the proposed changes to be acceptable.

3. ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 2, 1984

The following NRC personnel have contributed to this Safety Evaluation:
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