NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2 1	1 OF 0 2

TITLE (4)

Reactor Scram due to Turbine Stop Valve Fast Closure during Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 6	0 4	8 4	8 4	0 0 7	0 0	0 7	0 3	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 9 5		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Thomas L. Elton, Acting Superintendent of Regulatory Compliance	9 1 2 3 6 7 - 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
C	J C	I P S	B 0 7 0	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 0104 CDT on 6/4/84, Unit 1 had decreased load from 764 MWE to 7?0 MWE in order to perform the "MAIN TURBINE DAILY TEST" procedure (HNP-1-1051). When plant personnel released the test button for the Main Turbine number 2 Combined Intermediate Valve (i.e. one valve body with two stems, one stem "an Intermediate stop valve," the other stem" a control intercept valve") a scram due to Turbine Stop Valve fast closure Reactor Scram occurred at approximately 0121 CDT.

The cause of this event was postulated as being due to the disc dump not completely seating on the Main Turbine Number 2 Combined Intermediate Valve when the test button was released.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2	1 8 4	— 0 0 7	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day report is required by 10CFR 50.73(a)(2)(iv) due to this event resulting in automatic actuation of an ESF (RPS).

On 06/04/84 at approximately 0104 CDT, the unit was in steady state operation at 2312 MWT (approximately 95% power) following a turbine load reduction to perform the "MAIN TURBINE DAILY TEST" procedure (HNP-1-1051). When plant personnel released the test button for the Main Turbine number 2 Combined Intermediate Valve, a Turbine Stop Valve fast closure Reactor Scram (uplanned reactor scram) was received. There was no actual or potential safety consequences or implications. Personnel and equipment performed effectively resulting in a normal plant shutdown. However, personnel did report a discrepancy in that it appeared that the redundant Reactor High Pressure Scram Instrumentation (1B21-N023A-D) did not actuate properly. Further investigation revealed that the 1B21-N023A-D instruments exceeded the Tech. Specs. Reactor High Pressure Scram setpoint of less than or equal to 1045 PSIG by approximately 30 PSIG.

The cause of the Turbine Stop Valve fast closure Reactor Scram was postulated as being due to the disc dump not completely seating on the main turbine number 2 Combined Intermediate Valve when the test button was released. This would have resulted in the control hydraulic oil pressure for the Main Turbine Stop Valves and Combined Intermediate Valves decreasing rapidly. This rapid decrease would cause the Main Turbine Stop Valves to go from their full open position toward valve closure by more than 10% resulting in a Reactor Scram. The cause of the 1B21-N023A-D instruments being outside of the Tech. Specs. limits was due to faulty test equipment. The Ashcroft Digigauge used in performing calibration on 5/10/84 was found to be reading 40 PSIG lower than actual pressure.

The unit was brought to cold shutdown to investigate the scram and any discrepancies resulting from the scram. The Main Turbine Stop Valve accumulators and Combined Intermediate Valve accumulators were checked and found to be maintaining oil pressure within their acceptable range. The main turbine number 2 Combined Intermediate Valve was disassembled and no abnormalities were found. The Main Turbine number 2 Combined Intermediate Valve disc dump solenoid was replaced as part of the investigation. Also, HNP-1-1051 was revised on 06/08/84 per General Electric recommendation (TIL 969) to delete the daily testing of Main Turbine Stop Valves and Combined Intermediate Valves. The Reactor High Pressure Scram Instrumentation (1B21-N023A-D) was recalibrated immediately to within Tech. Specs. limit of less than or equal to 1045 PSIG. The equipment search performed on the faulty Ashcroft Digigauge revealed no discrepancies.

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444

Georgia Power

Edwin I. Hatch Nuclear Plant

July 3, 1984
GM-84-542

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1984-007. This report is required by 10CFR 50.73(a)(2)(iv).

H. C. Nix
General Manager

HCN/TLE/vlt

xc: R. J. Kelly
R. E. Conway
J. T. Beckham, Jr.
P. D. Rice
K. M. Gillespie
Superintendent of Regulatory Compliance
R. D. Baker
Control Room
Document Control

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