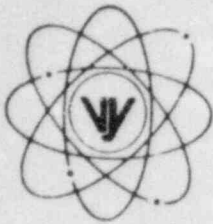


VERMONT YANKEE NUCLEAR POWER CORPORATION



RD 5, Box 169, Ferry Road, Brattleboro, VT 05301

FVY 84-72

REPLY TO:
ENGINEERING OFFICE

1671 WORCESTER ROAD
FRAMINGHAM, MASSACHUSETTS 01701
TELEPHONE 617-872-8100

June 28, 1984

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Office of Nuclear Reactor Regulation
Mr. Domenic B. Vassallo, Chief
Operating Reactors Branch No. 2
Division of Licensing

References: a) License No. DPR-28 (Docket No. 50-271)
b) Letter, VYNPC to USNRC, FVY 84-23, dated 3/15/84
c) Letter, VYNPC to USNRC, FVY 82-113, dated 10/21/82
d) Letter, USNRC to VYNPC, Nvy 83-262, dated 11/15/83

Dear Sir:

Subject: I&E Bulletin 80-11, Masonry Wall Design

By letter dated March 15, 1984 [Reference b)], we committed to providing you the summary results of our masonry wall reanalysis program. This program was initiated to close out remaining NRC questions relative to ensuring that certain masonry walls (in-fills) at our facility meet the criteria of I&E Bulletin 80-11. These walls are identified in Reference c).

We have recently completed our reanalysis program, the results of which indicate that nine of the ten in-fills in questions presently meet the masonry/mortar stress criteria of I&E Bulletin 80-11 and do not require any modifications. The remaining in-fill has two six-way anchor restraints bolted through it. The effects of seismic inertia for these two supports results in this in-fill being overstressed. To reduce the stress to within acceptable limits the two supports (RSW-H-219 and RSW-H-211) will be addressed as follows:

- o (RSW-H-219) An evaluation of the support and the in-fill, using refined piping loads, is scheduled to be completed by September 1984. Any necessary modifications will be completed during the 1985 Refueling Outage.
- o (RSW-H-211) This support will be removed from the in-fill and will be relocated and/or modified, as necessary during the 1985 Refueling Outage.

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Q PDR

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- o Because these supports are welded to the piping pressure boundary, any removal, relocation and/or modification activities could involve grinding and welding on the pipe surface, with a potential risk of compromising the integrity and availability of the piping system. For this reason, we believe it is most prudent to conduct these activities during an extended shutdown when the effect of such an occurrence would be minimized.

It should be noted that the calculations have been performed using the analytical procedures used in our original calculations. These procedures have been reviewed and approved by your consultant (Franklin Research Center), as documented in Reference d). In addition, the basis for the allowable stresses consisted of the ACI 531-79 Code, UBC Code (1979), and Appendix A to SRP 3.8.4. These calculations can be made available for your review at our Engineering Offices.

We trust that this information is deemed acceptable; however, should you have any questions in this matter, please contact us.

Very truly yours,

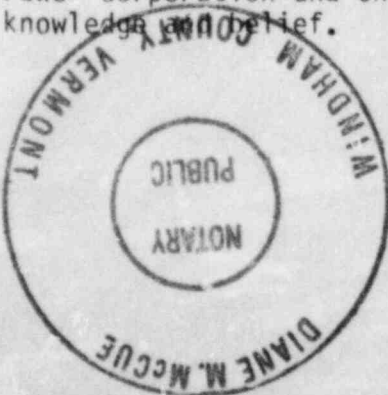
VERMONT YANKEE NUCLEAR POWER CORPORATION

W.P. Murphy
 W.P. Murphy
 Vice President and
 Manager of Operations

WPM/dm

STATE OF VERMONT)
)ss
 WINDHAM COUNTY)

Then personally appeared before me, Warren P. Murphy, who, being duly sworn, did state that he is Vice President and Manager of Operations of Vermont Yankee Nuclear Power Corporation, that he is duly authorized to execute and file the foregoing document in the name and on the behalf of Vermont Yankee Nuclear Power Corporation and that the statements therein are true to the best of his knowledge and belief.



Diane M. McCue
 Diane M. McCue Notary Public
 My Commission Expires February 10, 1987