NRC Form 366
(9/83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	1 OF 0 1

TITLE (4)

LOSS OF LOAD REACTOR TRIP

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 6	0 1	8 4	8 4	0 2 2	0 0	0 6	2 9	8 4		0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 9 2	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. G. HAYNES, STATION MANAGER	7 1 4 — 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	T T	J X	X 9 9 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0417 on June 1, 1984, during performance of Surveillance Procedure S023-3-3.34, "Turbine Overspeed Protection Valve Operability Tests," with Unit 3 at 92 per cent power, a failure in the Turbine Control System (EIIS System Identifier TT) caused all High Pressure Turbine Stop Valves (EIIS Component Identifier V) to close and a "Loss of Load" reactor trip. No systems or components malfunctioned during this event.

The turbine stop and governor valves are controlled by signals from three computing channels. If one channel's output deviates significantly from the remaining two channels, that channel's output will not be used in valve control. Although the exact cause of the trip could not be determined, it is suspected that one of the computing channels failed and the On Load Test Module lost power. These failures resulted in two of the three computing channels producing a "valve close" signal. All turbine stop and governor valves closed causing a "Loss of Load" reactor trip. The On Load Test Module power supply was replaced, and several loose circuit cards were reseated. In addition, S023-3-3.34 was revised to require an operator at the Turbine Governor Control Panel during valve testing to inform the Control Room if any off normal conditions are detected.

There are no reasonable or credible alternatives under which this event would have been more severe.

IE22
1/1

NRC Form 366
(9-83)

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

June 29, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-022
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73.a.2(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the actuation of the Reactor Protection System. Neither the health and safety of the public nor plant personnel were affected by this event.

If you require any additional information, please so advise.

Sincerely,

J G Haynes / [illegible]

Enclosure: LER 84-022

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

IE22
1/1