



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

1.0 Introduction

GPU Nuclear Corporation (GPU or the licensee) has installed Reactor Coolant System high point vents at Three Mile Island Nuclear Station, Unit No. 1 (TMI-1) pursuant to Item II.B.1 of NUREG-0737. In accordance with the NRC staff request in Generic Letter 83-37, dated November 1, 1983, GPU submitted proposed Technical Specifications for these vents in Technical Specification Change Request No. 137 transmitted by letter dated February 9, 1984. The purpose of this Safety Evaluation is to evaluate these proposed Technical Specification changes.

2.0 Evaluation

In accordance with 10 CFR 50.44 and NUREG-0737, Item II.B.1, high point vents are being installed at TMI-1. The vents are designed to remove noncondensable gas which might collect at the reactor system high points following an occurrence of inadequate core cooling. The vents are located on the reactor vessel head, each of the two hot legs and on the pressurizer. The vents are not required to mitigate any design basis accident but provide a defense in depth function. The vent design and testing program was previously approved by the NRC staff.

The proposed Technical Specifications regarding the high point vents provide for operation with the reactor critical for no more than 30 days with one vent path out of service and for no more than 72 hours with two or more vent paths out of service. The valves in each vent line are required to be tested for operability during each refueling outage. These requirements are consistent with our previous acceptance of the vent design and testing program and with Technical Specifications in place at other plants and are therefore acceptable.

3.0 Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area. We have determined that the amendment involves no significant increase in the amounts of

any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 21, 1984

The following NRC personnel have contributed to this Safety Evaluation:
W. Jensen, H. Silver.