NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
R. E. Ginna Nuclear Power Plant, Unit No. 1	0 5 0 0 0 2 4 4	1 OF 0 2

TITLE (4)
Automatic Actuation of Reactor Protection System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 5	3 0	8 4	8 4	0 0 7	0 0	0 6	2 9	8 4		0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 8 3	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
G. F. Larizza, Operations Manager	AREA CODE 3 1 5 5 2 4 - 4 4 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	T L	- E X C	W 1 2 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 30, 1984, while operating at approximately 83% power, an electrical generator trip occurred which in turn caused a trip of the turbine with subsequent reactor trip.

The cause of the trip was traced back to the electrical generator exciter, when a portion of neoprene gasket used in the exciter cooler was sucked into the air flow path and lodged into the rectifier area.

IE22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
R. E. Ginna Nuclear Power Plant Unit No. 1	0 5 0 0 0 2 4 4	8 4	— 0 0 7	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On May 30, 1984, while operating at approximately 83% power, an electrical generator trip occurred which in turn caused a trip of the turbine with subsequent reactor trip. After the reactor trip and initiation of auxiliary feedwater pumps, a cooldown and depressurization of the RCS to 530°F and 1950 psig followed. During this event, both steam generators (S/G) levels reached a level of < 16%. Valve AOV-427 "Letdown Isolation" had been left in the open position and did not close automatically on low pressurizer level signal. All other signals including safety injection would have closed the valve. Control Room Operator manually closed the valve. The procedure has been changed to specify leaving this valve in auto position rather than open.

The Control Room Operators manually isolated the Main Steam Line Isolation valves (MSIV) to limit the RCS cooldown. The B RCP was manually stopped to limit heat addition to the RCS. Systems were then returned to normal Hot Shutdown conditions.

An initial NRC notification of the event was made within four hours by telephone, in accordance with 10 CFR 50.72 B.2.II.

Subsequent post trip review revealed that the reactor protection system functioned properly, the initiating event being the loss of excitation of the electrical generator. The post trip review was presented to PORC on 6/1/84 and approval was given by the Plant Superintendent to restart the unit when the exciter work was completed.

The initiating event was a section of neoprene sponge rubber gasket used between the exciter cooler and exciter housing being sucked in the air flow path and lodged into the rectifier area of the exciter. This resulted in a phase to phase fault in the rotating rectifier.

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

June 29, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: LER 84-007, Automatic Actuation of the Reactor Protection System (RPS)

R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(iv) which requests a report of, "any event or conditions that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS)," the attached Licensee Event Report LER 84-007 is hereby submitted.

Very Truly Yours,

Roger W. Kober

Roger W. Kober

xc: U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

IE-22
1/1