SNUPPS
Standardized Nuclear Unit
Power Plant System

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Nicholas A. Petrick
Executive Director

February 1, 1984

SLNRC 84-0015 FILE: 0278
SUBJ: SNUPPS License Conditions

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Docket Nos. STN 50-482 and STN 50-483

Reference: SLNRC 83-0007, dated February 4, 1983; NUREG-0737 II.B.3, Post Accident Sampling Capability

Dear Mr. Denton:

License Condition 4 for the Callaway Plant (B.4 for Wolf Creek Generating Station) requires the periodic submittal of information regarding the adequacy of the computer-based, process noise method of sensor time response testing for the first SNUPPS plant utilizing this method. (See attached License Condition description.) Because of limitations associated with the process noise method, the SNUPPS Utilities do not plan to use this method as the primary means of sensor time response testing. The SNUPPS Utilities will be performing required sensor time response testing using conventional methods, for which much industry experience exists, as the primary means. The process noise methodology will be used for backup. The SNUPPS FSAR will be updated to reflect this information in a later revision. Therefore, the SNUPPS Utilities request that this License Condition be deleted from the licensing basis for the SNUPPS facilities.

License Condition 20 for the Callaway Plant (B.17 for Wolf Creek Generating Station) requires the demonstration of capability to promptly obtain reactor coolant samples in the event of an accident in which there is core damage. The referenced letter provided information concerning the Post-Accident Sampling System and sampling capabilities of the SNUPPS plants. Additionally, the SNUPPS Utilities will have the Post-Accident Sampling System installed and operational prior to exceeding 5% core power at each of the SNUPPS plants. Also, prior to operation at greater than 5% core power, the SNUPPS Utilities will provide to the NRC 1) the plant procedure which relates radionuclide species to estimated

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core damage using such parameters as hydrogen levels, reactor coolant system pressure, core exit thermocouple temperatures, and containment radiation levels, and 2) information demonstrating the applicability of procedures and instrumentation in the post-accident water chemistry and radiation environment. Training will be conducted per the plant qualification program in conjunction with participation in semi-annual emergency planning drills.

Based on the information provided in the referenced letter and the above commitments, the SNUPPS Utilities request that the License Condition regarding post-accident sampling capability be deleted.

Very truly yours,

Nicholas A. Petrick

MHF/nld7a18&19
Enclosure

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