

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>SURRY POWER STATION, UNIT 1</b>	DOCKET NUMBER (2) 0   5   0   0   0   2   8   0	PAGE (3) 1   OF   0   3
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TITLE (4)  
**OVERPRESSURE MITIGATING SYSTEM CHALLENGE**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)
0   6   0   1   8   4	8   4			0   1   3	0   0	0   6   2   9   8   4						0   5   0   0   0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)							
POWER LEVEL (10) 0   0   0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)				
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)				
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	<b>SPECIAL REPORT</b>				
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)					
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME <b>J. L. WILSON, STATION MANAGER</b>		AREA CODE	NUMBER
		8   0   4	3   5   7   -   3   1   8   4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 1, 1984, at 2343, the unit 1 reactor coolant system was solid and at 145°F and 325 psig. At this time, power operated relief valve (PORV) PCV-1455C cycled in response to an overpressure condition.

This event occurred while placing the charging system in service prior to unit startup, specifically, while opening MOV-1286B and C and MOV-1287A and B (Charging pump discharge valves). The MOV's were immediately closed and the pressure transient was terminated.

The cause of this event was personnel error due to improper preparation and verification of a tagging report that was prepared for flow element 1-SI-1940. Valve 1-SI-174 was returned to the open position in error providing a flow path from the charging pumps to the primary. The valve was closed and added to the preparation for solid plant operation procedure to verify it closed.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  SURRY POWER STATION, UNIT 1	DOCKET NUMBER (2)  0 5   0   0   0   2   8   0	LER NUMBER (6)			PAGE (3)		
		YEAR 8   4	SEQUENTIAL NUMBER —   0   1   3	REVISION NUMBER —   0   0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

1. Description of the Event

On June 1, 1984, at 2343, the unit 1 reactor coolant system was solid and at 145°F and 325 psig. At this time, power operated relief valve (PORV EIIS No. RV) PCV-1455C cycled in response to an overpressure condition.

This event occurred while placing the charging system in service prior to unit startup, specifically, while opening MOV-1286B and C (EIIS NO.V) and MOV-1287A and B (Charging pump (EIIS No. P) discharge valves). The MOV's were immediately closed and the pressure transient was terminated.

This event is being reported pursuant to the special reporting requirements of T.S.-3.1.G.3 and T.S.-6.6.4.g.

2. Probable Consequences

The overpressure mitigating system is intended to mitigate pressure transients when the primary system is less than 350°F. PORV PCV-1456 cycled 5 or 6 times to prevent the pressure from increasing beyond approximately 412 psig. The redundant PORV (PCV-1456) has a 10 psig higher lift setting than 1455C and it did not open.

The overpressure mitigating system performed as designed to prevent an overpressure condition, therefore an unreveiwed safety question was not created and the health and safety of the public were not affected.

3. Cause

Earlier during the day of the event, maintenance was completed on the Boron Injection Tank Bypass line flow element 1-SI-1940(EIIS No. FT). The tagging report used to isolate the flow element was improperly prepared such that the normally closed 3/4" bypass valve, 1-SI-174, was returned to the open position. The valve should have been left closed. This valve alignment providee a path from the charging pumps to the primary system when the charging pump discharge MOV's were open. The cause of this event was personnel error due to improper preparation of the tagging report. Independent review conducted by an SRO failed to detect the improper "return to" position specified.

4. Immediate Corrective Actions

The charging pump discharge MOV's were immediately shut to terminate the pressure transient.

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FACILITY NAME (1)  SURREY POWER STATION, UNIT 1	DOCKET NUMBER (2)  0 5 0 0 0 2 8 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 1 3	0 0	0 3	OF	0 3

NOTE: If more space is required, use additional NRC Form 366A's (17)

5. Additional Corrective Action

Normal pressure control devices were used to return the primary system pressure to 325 psig.

Though the trouble shooting efforts of the shift supervisor, valve 1-SI-174 was found open and was closed.

The procedure for preparing for solid plant OPS will be revised to require verifying 1-SI-174 in the closed position.

6. Action Taken to Prevent Recurrence

Valve 1-SI-174 was added to operating procedure OP-1E (Containment Integrity Checklist) to reduce the chances of additional PORV lifts.

7. Generic Implications

This is considered as a random event.

# Vepco

VIRGINIA ELECTRIC AND POWER COMPANY  
Surry Power Station  
P. O. Box 315  
Surry, Virginia 23883

JUN 29 1984

Serial No: 84-027

Docket No: 50-280

License No: DPR-32

U.S. Nuclear Regulatory Commission  
Document Control Desk  
016 Phillips Building  
Washington, D. C. 20555

Gentlemen:

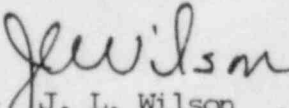
Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

84-013-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

  
J. L. Wilson  
Station Manager

Enclosure

cc: Mr. James P. O'Reilly  
Regional Administrator  
Suite 2900  
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11