

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	PAGE (3) 1 0 7 0 8
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TITLE (4)
DOSE EQUIVALENT IODINE LIMITS EXCEEDED

EVENT DATE (5)			LER NUMBER (C)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
06	01	84	84	023	000	06	28	84		0 5 0 0 0 0
										0 5 0 0 0 0

OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME J. G. HAYNES, STATION MANAGER	TELEPHONE NUMBER
	AREA CODE: 7 1 4 NUMBER: 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO
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EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the Reactor Coolant System specific activity.

On June 1, 1984, the reactor tripped at 0417 during the monthly turbine overspeed protection test. Following the reactor trip, at 0620, analysis of a Reactor Coolant System (RCS) sample indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 0130 on June 3, 1984.

On June 11, 1984, the reactor tripped at 1817 due to a Core Protection Calculator generated low Departure from Nucleate Boiling Ratio (DNBR). Following the reactor trip, at 2100, RCS sample analysis indicated that RCS specific activity exceeded 1.0 microcurie/gram DE I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 0230 on June 13, 1984.

The two events were indications of iodine spiking. We will continue to monitor and evaluate primary coolant activity. No further corrective action is planned.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 2 3	- 0 0	0 2	OF	0 8

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the Reactor Coolant System (RCS) specific activity.

On June 1, 1984, the reactor tripped at 0417 during the monthly turbine overspeed protection test (see LER 84-022). Following the reactor trip, at 0620, RCS sample analysis indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 0130 on June 3, 1984.

On June 11, 1984, the reactor tripped at 1817 due to a Core Protection Calculator generated low Departure from Nucleate Boiling Ratio (DNBR) (see LER 84-024). Following the reactor trip, at 2100, RCS sample analysis indicated that RCS specific activity exceeded 1.0 microcurie/gram DE I-131. RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 by purification flow at 0230 on June 13, 1984.

The two events were indications of iodine spiking. Similar occurrences were previously reported in LER 83-111, LER 84-005, LER 84-013, and LER 84-015. We will continue to monitor and evaluate primary coolant activity. No further corrective action is planned. There was no impact on the health and safety of the public associated with these events.

Additional information, required by LCO 3.4.7, Action Statement 'd', is provided on the following pages. Although the unit has a degasification path which operates continuously and takes pressurizer steam, condenses it and directs it to Liquid Radwaste, degassing operation history is not applicable, because this system reduces the noble gas content of the RCS but has no effect on iodine.

LICENSEE EVENT REPORT (LER)
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		YEAR 8 4	SEQ. NUMBER - 0 2 3	REV. NUMBER - 0 0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CLEANUP FLOW HISTORY

<u>PERIOD</u>	<u>AVERAGE CLEANUP FLOW (GPM)</u>
5/30/84, 0600 to 5/30/84, 2000	69.78
5/30/84, 2000 to 5/30/84, 2200	85*
5/30/84, 2200 to 6/01/84, 0600	80.72
6/09/84, 2100 to 6/10/84, 2400	81.5
6/10/84, 2400 to 6/11/84, 0600	85*
6/11/84, 0600 to 6/11/84, 2100	83.5

*Hourly cleanup flow data not available. Figure used is taken from average flow with two charging pumps in operation.

REACTOR POWER HISTORY

<u>PERIOD</u>	<u>REACTOR POWER</u>
5/30/84, 0600 to 5/30/84, 2400	100% Rated Power
6/01/84, 0100	93%
6/01/84, 0200 to 6/01/84, 0400	92%
6/01/84, 0500 to 6/01/84, 0600	0%
6/09/84, 2100 to 6/11/84, 1800	100%
6/11/84, 1900 to 6/11/84, 2100	0%

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		YEAR 8 4	SEQ. NUMBER - 0 2 3	REV. NUMBER - 0 0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSIS

<u>DATE</u>	<u>TIME</u>	<u>DE I-131 MICROCURIES/GRAM</u>
6/01/84	0620	1.544
6/01/84	1020	1.990
6/01/84	1330	1.755
6/01/84	1740	1.783
6/01/84	2130	1.502
6/02/84	0130	1.235
6/02/84	0600	1.652
6/02/84	0930	1.443
6/02/84	1330	1.413
6/02/84	1730	1.323
6/02/84	2130	1.170
6/03/84	0130	0.940
6/11/84	2100	2.61
6/12/84	0120	2.48
6/12/84	0455	1.93
6/12/84	0855	1.40
6/12/84	1045	1.24
6/12/84	1443	1.77
6/12/84	1855	1.56
6/12/84	2230	1.194
6/13/84	0230	0.897

The total time with the DE I-131 above 1.0 microcuries/gram for this event was 72.666 hours. Cumulative time with DE I-131 above 1.0 microcuries/gram for last 12-month period is 258.266 hours.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME IS SAN ONOFRE NUCLEAR GENERATING STATION UNIT 3	EVENT NUMBER IS 05000362	LER NUMBER IS			PAGE IS	
		YEAR	SEQUENTIAL NUMBER	REVISED NUMBER		
		84	023	00	06	OF 08

NOTE: If more space is required, use additional NRC Form 1084s (17)

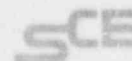
BATCH NUMBER	BATCH NAME	AVERAGE EXPOSURE (EMD/FT)
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000004	000004	0.00
000005	000005	0.00
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MAXIMUM INTENSIFIED ASSEMBLY EXPOSURE IS 0.06877084 RY/FT IN ASSEMBLY 118
MAXIMUM ASSEMBLY EXPOSURE IS 0.04129014 RWD/FT OCCURRING AT 31.18 FT OF THE CORE HEIGHT IN ASSEMBLY 118
CORE AVERAGE EXPOSURE IS 0.02587824 RWD/FT
EQUAL TO 107.09 EFPO

BATCH NUMBER BATCH NAME AVERAGE EXPOSURE (EMD/FT)

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000002 000002 0.00
000003 000003 0.00
000004 000004 0.00
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Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

June 28, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-023
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the Reactor Coolant System specific activity. There was no impact on the health and safety of the public associated with these events.

If you require any additional information, please so advise.

Sincerely,

J. G. Haynes / [Signature]

Enclosure: LER No. 84-023

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)
J. B. Martin (Regional Administrator, USNRC Region V)
Institute of Nuclear Power Operations (INPO)

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