

Document Control Desk
LER No. 83-136, Revision 1
Page Two
June 29, 1984

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On November 28, 1983, the Plant was shutdown for post-steam generator modification eddy current testing. The Plant was in Mode 5 with Train "A" Residual Heat Removal (RHR) System in service.

A modification was scheduled to be performed on Engineered Safety Features (ESF) inverter, XIT-5901, while the Plant was shut down. XIT-5901 is the normal power source for APN-5901, a 120 VAC vital instrument power supply panel for ESF instrumentation. In order to perform the modification on the inverter, APN-5901 was transferred to its alternate power supply, APN-1FA.

Approximately five minutes had elapsed when the Operator at the Controls (OATC) noticed RCS temperature increasing. Upon checking the RHR valve line-up, the OATC discovered XVG-8701A, "RHR Suction Isolation Valve," from the RCS closed. The valve was immediately reopened to restore Train "A" RHR to service. RCS temperature reached 124°F maximum. XVG-8701A was reopened within approximately five minutes of its closure.

Due to plant conditions and the short duration of the event, there were no resulting adverse consequences. The loss of the Train "A" RHR System necessitated entry into Action Statement (b) of Technical Specification 3.4.1.4.2. Technical Specification compliance was maintained at all times.

CAUSE AND CORRECTIVE ACTIONS

Pressure transmitter, PT-403, receives loop power from APN-5901 via instrument power panel XPN-7001. PT-403 is the pressure instrument for XVG-8701A. In transferring APN-5901 from normal to alternate power supply, the ESF instrument bus powered by the power panel was momentarily de-energized which created a dead bus transfer. The power transient resulted in the spiking of PT-403 and subsequent closure of XVG-8701A.

This condition returned to normal operation upon completion of the electrical line-up.

An APN power distribution list will be generated by December 31, 1984, to enhance operator anticipation of plant instrumentation responses during transients of this type which may occur in the future.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

June 29, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 83-136, Revision 1

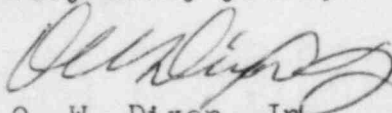
Dear Sir:

Please find attached Revision 1 to Licensee Event Report #83-136 for the Virgil C. Summer Nuclear Station. This Thirty Day Report was previously submitted on December 27, 1983, in accordance with the requirements of Technical Specification 6.9.1.13.(b) and reported an entry into Action Statement (b) of Technical Specification 3.4.1.4.2.

This Revision provides an updated completion schedule for an APN power distribution list. The Licensee has initiated the development of the list; however, a comprehensive list will not be completed until December 31, 1984. The completion of this commitment has been delayed because of the complexity of the task and manpower limitations.

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

CJM:OWD/dwf
Attachment

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11