



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106  
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated April 11, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraphs 2.C.(2) and 2.C.(35) of Facility Operating License No. NPF-11 and is hereby amended to read as follows:

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\*Page 16a is attached, for convenience, for the composite license to reflect this change.

2.C.(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 106, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

2.C.(35) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated April 11, 1995, shall be extended to April 5, 1996, to coincide with the Unit 1 seventh refueling outage schedule. The extended interval shall not exceed a total of 25.1 months for 18 month surveillances.

3. This amendment is effective upon date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert M. Latta, Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License Page 16a

Date of Issuance: September 27, 1995

2.C.(34) Deleted.

2.C.(35) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated April 11, 1995, shall be extended to April 5, 1996, to coincide with the Unit 1 seventh refueling outage schedule. The extended interval shall not exceed a total of 25.1 months for 18 month surveillances.



UNITED STATES  
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COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92  
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated April 11, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraphs 2.C.(2) and 2.C.(19) of Facility Operating License No. NPF-18 and is hereby amended to read as follows:

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\*Pages 8 and 9 are attached, for convenience, for the composite license to reflect this change.

2.C.(2) Technical Specifications and Environmental Protection Plan

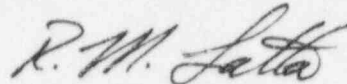
The Technical Specifications contained in Appendix A, as revised through Amendment No. 92, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

2.C.(19) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated April 11, 1995, shall be extended to April 5, 1996, to coincide with the Unit 1 seventh refueling outage schedule. The extended interval shall not exceed a total of 25.1 months for 18 month surveillances.

3. This amendment is effective upon date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert M. Latta, Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License Pages 8 and 9

Date of Issuance: September 27, 1995

(17) Initial Test Program (Section 14, SER, SSER #7)

The licensee shall conduct the initial startup test program (set forth in Section 14 of the licensee's Final Safety Analysis Report, as amended) without making any modifications of this program unless such modifications are in accordance with the provisions of 10 CFR Section 50.59. In addition, the licensee shall not make any major modifications to this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- (a) Elimination of any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended, as being essential;
- (b) Modification of test objectives, methods or level 1 acceptance criteria for any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended, as being essential;
- (c) Performance of any test at a power level different from that described in the program; and
- (d) Failure to complete any tests included in the described program (planned or scheduled) for power levels up to the authorized power level.

(18) NUREG-0737 Conditions (Section 22.2)

The licensee shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2, 3, 4, 5, and 7, NUREG-0519.

- (a) Emergency Response Capability (I.D.1, SER, SSER #1, SSER #5)

The licensee shall correct the design deficiencies for the control room and complete the other related emergency response capabilities as required by Attachment 2 of this license.

(b) Direct Indication of Safety/Relief Valve Position (II.D.3, SER, SSER #2 and Section 3.10, SSER #7)

Prior to completion of the startup test program, the licensee shall qualify the safety/relief valve position indicator.

(c) Instrumentation for Detection of Inadequate Core Cooling (II.F.2, SER, SSEP #1, SSER #5)

The licensee shall implement the NRC staff's requirements regarding upgrading the liquid level instrumentation or inclusion of additional instrumentation for detecting inadequate core cooling based on the NRC staff's review of the BWR Owners Group Reports SLI-8211 and SLI-8218 and the licensee's plant-specific evaluation report addressing the recommendations of the BWR Owners Group reports. Any required modifications shall be completed on a schedule acceptable to the NRC staff.

(d) Modification of Automatic Depressurization System Logic - Feasibility for Increased Diversity for Some Event Sequences (II.K.3.18, SER, SER #1, SSER #3, SSER #5)

Prior to startup after the first refueling outage, the licensee shall:

(i) Install modifications to the Automatic Depressurization system described in the licensee's letter dated July 1, 1983. The final circuit diagrams and an analysis of the bypass timer time delay shall be submitted for NRC staff review and approval prior to installation.

(ii) Incorporate into the Plant Abnormal Procedures the usage of the inhibit switch; and

(iii) Modify the Technical Specifications to provide the bypass timer and manual inhibit switch.

(19) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated April 11, 1995, shall be extended to April 5, 1996, to coincide with the Unit 1 seventh refueling outage schedule. The extended interval shall not exceed a total of 25.1 months for 18 month surveillances.