

FEB 21 1992

Docket No. 50-333

New York Power Authority
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. Radford Converse
Resident Manager
Post Office Box 41
Lycoming, New York 13093

Dear Mr. Converse:

SUBJECT: FIRE PROTECTION DOCUMENTS FOR FITZPATRICK

As discussed in the February 7, 1992, working meeting held in our Region I office, your staff was advised that, as a result of the fire protection issues raised during the Diagnostic Evaluation Team (DET) inspection, a Special Team Inspection will be conducted at your FitzPatrick facility in the area of fire protection. This inspection will be performed on March 9-20, 1992, and be composed of personnel from several NRC offices and Brookhaven National Laboratory.

The purposes of this inspection are to assess the status of corrective actions with regard to the fire protection issues you took following the DET to evaluate the adequacy of your plant fire protection program and its implementation, and to assess FitzPatrick's compliance with Sections III.G, III.J, and III.L of Appendix R, to 10 CFR 50.

In order for us to conduct this type of comprehensive inspection, we will need to review in advance documents concerning the 10 CFR 50, Appendix R and fire protection programs at your FitzPatrick facility. Please provide to us three sets of the following documents. Once these documents have been retrieved and are available for review, please contact us for further instructions on their delivery to us.

1. The FitzPatrick's Fire Protection Program and Fire Hazards Analysis.
2. Fire Protection Program implementing procedures (e.g., administrative controls, surveillance testing, and fire brigade).
3. Fire brigade training program and pre-fire plans.
4. Engineering evaluations, construction and design details, and supporting fire tests associated with fire resistive barrier assemblies used at FitzPatrick.

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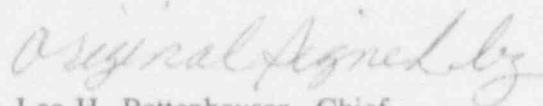
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5. Appendix R, safe shutdown systems and separation analysis.
6. Appendix R, alternative shutdown analysis.
7. Piping and instrumentation (flow) diagrams highlighting the components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.
8. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.
9. Color-coded, marked-up electrical raceway drawings which identify the routing of power and control circuits for those plant systems necessary to achieve and maintain safe shutdown.
10. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.
11. Plant layout drawings which identify the general location of the Appendix R, emergency lighting units.
12. Associated circuit analysis performed to assure the shutdown functions and alternative shutdown capability are not affected by hot shorts, shorts to ground, or open circuits (e.g., analysis of associated circuits for spurious equipment operations, common enclosure, and common bus).
13. Plant operating procedures which would be used and describe shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room procedures which would be used to implement alternative shutdown capability in the event of a fire in either the control or cable spreading room.
14. Surveillance testing procedures for alternative shutdown capability.
15. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the Appendix R breaker coordination analysis.

In addition to the information requested above, copies of any other analysis and/or evaluations which demonstrate compliance to Sections III.G, III.J, and III.L should be available on site for review during this inspection. Appropriate personnel, knowledgeable with respect to those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room, the fire protection programs and its implementation, and the electrical and systems aspects of the Appendix R analysis, should be available at the site during the inspection.

Your cooperation with us during this inspection will be appreciated. Should you have any questions concerning this letter and our information request, Mr. Sadan Pullani, the Team Leader will be glad to discuss them with you. Mr. Pullani can be reached at (301) 492-4043. If Mr. Pullani cannot be reached, alternate contacts would be Mr. Norm Blumberg at (215) 337-5199 or Mr. Patrick Madden at (301) 504-2854.

Sincerely,



Lee H. Bettenhausen, Chief
Operations Branch
Division of Reactor Safety

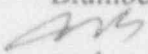
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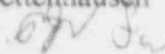
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