

June 15, 1984  
(NMP2L 0083)

Mr. R. W. Starostecki, Director  
U.S. Nuclear Regulatory Commission  
Region I  
Division of Project and Resident Programs  
631 Park Avenue  
King of Prussia, PA 19406

Re: Nine Mile Point Unit 2  
Docket No. 50-410

Dear Mr. Starostecki:

Enclosed is a final report in accordance with 10CFR50.55(e) for the problem concerning feedwater piping spools (55(e)-84-05). This problem was reported in a telephone conversation between T. Loomis (Nine Mile Point Unit 2 Licensing) and S. Collins of your staff on February 1, 1984. An interim report was submitted via our letter dated March 5, 1984.

Very truly yours,

*C. V. Mangan*

C. V. Mangan  
Vice President

Nuclear Engineering & Licensing

TRL:ja  
Enclosure  
xc: Director of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

R. A. Gramm, Resident Inspector

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NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT UNIT 2  
DOCKET NO. 50-410

Final Report for a Problem  
Concerning Feedwater Piping Spools  
(55(e)-84-05)

Description of the Problem

During a Stone & Webster Engineering Corporation audit of ITT Grinnell, it was observed that the material used in the fabrication of some feedwater piping spools did not meet all the impact testing requirements of ASME Section III. Certified Material Test Reports for the material in question did not document the test coupon heat treatment duration and temperature as required by ASME Section III, NB-2211. Further inquiry revealed that the heat treatment of the test coupon was in fact not performed. Heat treatment of the test coupons is required to ensure that the coupons reflect the material conditions of the spool piece. Eight feedwater pipe spools were affected.

The material test coupons with the same heat lot numbers as those of the eight feedwater pipe spools have since been heat treated and impact tested. Impact test results of the coupons indicated that seven spools meet the requirements of ASME Section III. The coupon of the remaining spool piece did not meet the impact test lateral expansion criterion of ASME Section III.

Analysis of Safety Implications

A detailed investigation of the spool piece that did not meet lateral expansion criterion, to determine its suitability for use, was not performed. Instead, this spool piece will be replaced with one meeting ASME III requirements. Had the investigation indicated that the spool piece would not have performed its intended design function, a possibility exists that the integrity of the reactor coolant pressure boundary could have been jeopardized. Therefore, if this problem were to have remained uncorrected, it could have adversely affected the safe operation of the plant.

Corrective Action

The spool piece that did not meet ASME III requirements will be replaced by September 14, 1984.

SEP 14 1984

SECTION 1