TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER BROWNS FERRY NUCLEAR PLANT

MONTHLY OPERATING REPORT TO NRC May 1, 1984 - May 31, 1984

DOCKET NUMBERS 50-259. 50-260, AND 50-296 LJCENSE NUMBERS DPR-33, DPR-52, AND DPR-68

8407020378 840611 PDR ADDCK 05000259 R PDR

Submitted by: <u>ASymbolic</u> 6., Plant Manager

### Operations Summary

# May 1984

The following summary describes the significant operation activities during the reporting period. In support of this summary, a chronological log of significant events is included in this report.

There was one reportable occurrence and two revisions to previous reportable occurrences reported to the NRC during the month of May.

Unit 1

There were no scrams on the unit during the month. Unit 2

There were no scrams on the unit during the month. Unit 3

The unit was in cold shutdown the entire month for the unit's end-of-cycle 5 refueling outage.

Prepared principally by B. L. Porter.

# Operations Summary (Continued)

May 1984

# Fatigue Usage Evaluation

The cumulative usage factors for the reactor vessel are as follows: Location Usage Factor Unit 1 Unit 2 Unit 3 Shell at water line 0.00486 0.00598 0.00403 Feedwater nozzle 0.28849 0.21103 0.15429 Closure studs 0.23477 0.17236 0.13233 NOTE: This accumulated monthly information satisfies Technical

Specification Section 6.6.A. 17.B(3) reporting requirements.

### Common System

Approximately 4.95E+05 gallons of waste liquids were discharged containing approximately 9.05E-02 curies of activities.

# Operations Summary (Continued)

# May 1984

# Refueling Information

Unit 1

Unit 1 is scheduled for its sixth refueling beginning on or about February 8, 1985 with a scheduled restart date of August 27, 1985. This refueling will involve loading 8x8R (retrofit) fuel assemblies into the core, replacing recirculation piping, work on "A" and "B" low-pressure turbine, upgrade hangers and anchors, and environmentally qualify instrumentations.

There are 764 fuel assemblies in the reactor vessel. The spent fuel storage pool presently contains 252 EOC-5 fuel assemblies, 260 EOC-4 fuel assemblies; 232 EOC-3 fuel assemblies; 156 EOC-2 fuel assemblies; and 168 EOC-1 fuel assemblies. The present fuel pool capacity is 3,471 locations.

# Unit 2

Unit 2 is scheduled for its fifth refueling beginning on or about August 24, 1984 with a scheduled restart date of January 23, 1985. This refueling outage will involve loading additional 8X8R (retrofit) fuel assemblies into the core, finishing the torus modification, turbine inspection, finishing piping inspection, finishing TMI-2 modifications; possaccident sampling facility tie-ins, core spray change-out, and feedwater sparger inspection.

There are 764 fuel assemblies in the reactor vessel. At the end of the month there were 248 EOC-4 fuel assemblies, 353 EOC-3 fuel assemblies, 156 EOC-2 fuel assemblies, and 132 EOC-1 fuel assemblies in the spent fuel storage pool. The present available capacity of the spent fuel pool is 601 locations.

# Operations Summary (Continued)

May 1984

### Unit 3

Unit 3 shutdown for its fifth refueling outage on September 7, 1983, with a scheduled restart date of August 1, 1984. This refueling involves loading 8X8R (retrofit) assemblies into the core, finishing the torus modifications, postaccident sampling facility tie-in, core spray change-out, finishing TMI-2 modifications, turbine inspection, piping inspections for cracks, and changeout of jet pump hold-down beams.

There are 0 fuel assemblies presently in the reactor vessel. There are 248 new fuel assemblies, 764 EOC-5 fuel assemblies, 280 EOC-4 fuel assemblies, 124 EOC-3 fuel assemblies, 144 EOC-2 fuel assemblies, and 208 EOC-1 fuel assemblies in the spent fuel storage pool. The present available capacity of the spent fuel pool is 150 locations.

Date	Time	Event
		Unit 1
5/1	0001	Reactor thermal power at 100-percent (%), maximum flow, rod limited.
5/5	0010	Commenced reducing thermal power for control valve test and SI's.
	0100	Reactor thermal power at 94% for turbine control valve test and SI's.
	0330	Turbine control valve test and SI's complete, commenced power ascension.
	0400	Reactor thermal power at 100%, maximum flow, rod limited.
5/12	0045	Commenced reducing thermal power for turbine control valve test and SI's.
	0200	Reactor thermal power at 91% for turbine control valve test and SI's.
	0325	Turbine control valve test and SI's complete, reducing thermal power for control rod pattern adjustment.
	0430	Reactor thermal power at 83% for control rod pattern adjustment.
	0500	Control rod pattern adjustment complete, commenced power ascension.
	0630	Commenced PCIOMR from 95% thermal power.
	0735	Commenced reducing thermal power from 97% for SI 4.3.A.2 (Control Rod Drive Exercise).
	0800	Reactor thermal power at 94% for SI 4.3.A.2.
	1015	SI 4.3.A.2 complete, commenced PCIOMR from 94% power.
	1300	Reactor thermal power at 100% maximum flow, rod limited.
5/13	0315	Commenced reducing thermal power from 100%, at load dispatchers request, load not needed.
	0500	Reactor thermal power at 82%, load not needed.
	0630	Commenced power ascension from 82% thermal power.
	0645	Commenced PCIOMR from 95% thermal power.
	1000	Reactor thermal power at 100%, maximum flow, rod limited.
5/17	1420	Commenced reducing thermal power for a control rod pattern adjustment.
	1446	Reactor thermal power at 93% for control rod pattern adjustment.
	1455	Control rod pattern adjustment complete, commenced power ascension.
	1500	Commenced PCIOMR from 95% thermal power.

Date	Time	Event
		Unit 1 (Continued)
5/17 (Cont)	2105	Stopped PCIOMR at 99% power and commenced reducing power to remove "B" string low-pressure heaters from service for maintenance.
	2220	"B" string low-pressure heater out-of-service for maintenance, reactor power at 65% and reducing.
5/18	0100	Reactor thermal power at 64%, "B" string low-press re heaters limited.
	0500	Reactor thermal power at 65%, "B" string low-pressure heaters limited.
	0545	"C" reactor feed water (RFW) pump out-of-service for maintenance, reactor power at 65%.
	0800	Reactor thermal power at 66%, "B" string low-pressure heaters and "C" REW pump limited.
	1000	Reactor thermal power at 62%, "B" string low-pressure heaters and "C" REW pump limited.
	1115	Commonood increasing thermal power from 58%
	1500	Reactor thermal power at 64%, "B" string low-pressure heaters and "C" RFW pump limited.
	2215	Commenced reducing thermal power due to partial isolation on Reactor Water Cleanup (RWCU), Traversing Incore Probes System (TIPS), Drywell Floor Drain (DWFD) and Equipment Drain (ED) sump pumps, drywell air conditioning, drywell delta pressure air compressor, Pressure System Charging (PSC) supply valves, 256 continuous air monitors (CAMs) and H <sub>2</sub> O <sub>2</sub> analyzers, IP-2 declared
	2225	Isolation reset, electricians found a loose wire on Primary Containment Isolation System (PCIS) reset relay.
	2240	Shutdown and IP-2 terminated, reactor thermal power at 60%.
	2300	Commenced power ascension from 60% power.
•	2343	"B" string low-pressure heaters back in service at 63% thermal power.
5/19	0700	Reactor thermal power at 75% holding due to "A" string high-pressure heater problems, and "C" RFW pump.
	1245	Commenced reducing thermal power from 75% thermal power to remove "A" string high-pressure heaters from service for maintenance.
	1300	"A" string high-pressure heaters and "C" RFW pumps out- of-service for maintenance, reactor thermal power at 71%.
	1600	Reactor thermal power at 70% for maintenance on "A" string high-pressure heaters, and "C" RFW pump.

Date	Time	Event	
		Unit 1 (Continued)	
5/19	1925	"C" RFW pump back in service, commenced power ascension.	
(Cont)	1930	"B" RFW pump taken out-of-service for maintenance, power ascension continues from 72%.	
	1955	"A" high-pressure heaters back in service, power ascension continues from 73%.	
	2300	Reactor thermal power at 75%, "B" RFW pump limited.	
5/20	0740	"B" RFW pump back in service, reactor power at 75%.	
	0755	"A" RFW pump out-of-service for maintenance, reducing thermal power.	
	1200	Reactor thermal power at 74%. "A" RFW pump limited.	
	2045	"A" RFW pump back in service, commenced power ascension.	
	2200	Commenced PCIOMR from 84% thermal power.	
5/21	2100	Reactor thermal power at 100%, maximum flow, rod limited.	
	2300	Reactor thermal power at 99%, "R" factor out-of-limits.	
5/22	0500	Reactor thermal power at 98%, "R" factor out-of-limits.	
	1200	Reactor thermal power at 97%, "R" factor out-of-limits.	
	1230	Commenced power ascension from 97% thermal power.	
	1300	Reactor thermal power at 100%, maximum flow, rod limited.	
5/23	0200	Reactor thermal power at 99%, "R" factor out-of-limits.	
	1030	Commenced power ascension from 99% power.	
	1100	Reactor thermal power at 100% maximum flow, rod limited.	
	1220	Reduced thermal power to 99%, "B" recirculation pump	
	1333	Commenced power ascension from 99% power.	
	1400	Reactor thermal power at 100%. maximum flow. rod	
		limited.	
E/06	0015	Commenced reducing thermal power for turbine control	
5/20	0015	valve test and SI's.	
	0200	Reactor thermal power at 94% for turbine control valve test and SI's.	
	0300	Turbine control valve test and SI's complete, commenced	
	0400	Commenced PCIOMR from 99% thermal power.	
	0500	Reactor thermal power at 100%, maximum flow, rod limited.	
5/31	2400	Reactor thermal power at 100%, maximum flow, rod limited.	

Date	Time Event			
		Unit 2		
5/1	0001	Reactor thermal power at 61%, derated to extend full cycle.		
5/2	0728	Reactor thermal power at 60%, derated to extend full cycle.		
5/4	0001	Reactor thermal power at 61%, derated to extend full cycle.		
5/10	0001	Reactor thermal power at 60%, derated to extend fuel cycle and administrative hold because all Automatic Depressurization System (ADS) relief valve cables are routed through the same cable tray.		
5/11	0001	Reactor thermal power at 59%, derated to extend full cycle and administrative hold.		
5/30	0900	Reactor thermal power at 60%, derated to extend full cycle and administrative hold.		
5/31	2400	Reactor thermal power at 60%, derated to extend full		

Date	Time	Event	
		Unit 3	
5/1	0001	End-of-cycle 5 refuel outage continues.	
5/31	2400	End-of-cycle 5 refuel outage continues.	

10 AVERAGE DAILY UNIT POWER LEVEL

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DOCKET NO.	50-259	
DOCKET NO. UNIT DATE COMPLETED BY	1	
	6/1/84	
	Ted Thom	
TELEPHONE	205/729-0834	

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
075	17	1039
-079	18	645
1076	19	738
1090	20	796
1054	21	994
1078	,,,	1058
1073		1070
1076	25	1059
1075	24	1066
1075	25	1048
1073	26	1082
1042	12/	1050
1043	20	1062
1064	29	1068
1080	.30	1073
1070	31	

# INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawart.

(11/77)

# 11 AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-260	
UNIT	2	
DATE	6/1/84	
OMPLETED BY	Ted Thom	
TELEPHONE	205/729-0834	

MONTH	nay 1904		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	617	17	607
2	615	18	593
1	611	10	596
	633	14	597
*	627	20	
5		21	592
6	591		595
7	618		596
8	619		591
9	620	24	592
	603	25	587
10	600	26	566
п .		27	
12	601	28	622
13	600	10	593
14	589		607
	502	.30	606
16	597	31	
10 .			

# INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-296	
UNIT	3	
DATE	6/1/84	
COMPLETED BY	Ted Thom	
TELEPHONE	205/729-0834	

MONT	нМау 1984		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1		17	4
• 2	-3		-3
3	-3	19	-2
4	-3	10	-2
5	-2	20	-3
6	-3		-3
-	-3	,1	-3
	-4		-4
	-/1	-4	-5
	-4	25	
10		26	
11	-4	27	-5
12		28	-4
13	-2	- 10	-4
14	-3	10	-4
	-4		-4
16	-4	41	

# INSTRUCTIONS

On this format, list the sverige daily multiples of here of Mwe Net for each day in the reporting month. Compute to the nearest whole meraway.

(1177)

### OPERATING DATA REPORT

# DOCKET NO. 50-259 DATE 6/1/84 COMPLETED BY Ted Thom TELEPHONE 2057729-0834

# OPERATING STATUS

I Unit Nume Browns Ferry - 1	Notes
2 Reporting Period: May 1984	
3. Licensed Thermal Power (MWt):3293	
4. Nameplate Rating (Gross Mwe):1152	
5. Design Electrical Rating (Net MWe): 1065	
6. Maximum Dependable Capacity (Gross MWe):1098.4	
7. Maximum Dependable Capacity (Net MWe):1065	_
i maximum rependable capacity (net aine).	

8 If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

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	۰.	8.		
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9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_N/A

10. Reasons For Restrictions, If Any:

# N/A

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744	3,647	86,209
12. Number Of Hours Reactor Was Critical	744	3,405 28	53,211.40
13. Reactor Reserve Shutdown Hours	0	225.40	6,010.42
14. Hours Generator On-Line	744	3,309.38	52,027.02
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2,357,182	9,920,114	148,477,793
17 Gross Electrical Energy Generated (MWH)	786,510	3,332,590	48,978,210
18. Net Electrical Energy Generated (MWH)	767,289	3,248,425	47,573,752
19. Unit Service Factor	100	90.7	60.3
20. Unit Availability Factor	100	90.7	60.3
21 Unit Capacity Factor (Using MDC Net)	96.8	83.6	51.8
22 Unit Capacity Factor (Using DER Net)	96.8	83.6	51.8
23. Unit Forced Outage Rate	0	8.4	23.0
	and the second se	title Arran Merican Arran and analytic and an and a first data	

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:			
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved	
INITIAL CRITICALITY			

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

## **OPERATING DATA REPORT**

DOCKET NO	50-260
DATE	6-1-84
COMPLETED BY	Ted Thom
TELEPHONE	205/729-0834

#### OPERATING STATUS

1 Unit Name Browns Ferry - 2	Notes
2 Reporting Period: May 1984	
3. Licensed Thermal Power (MWt):3293	표정 잘 것 같은 것 같은 것 같은 것 같은 것 같이 많이
4. Nameplate Rating (Gross MWe): 1152	
5. Design Electrical Rating (Net MWe): 1065	
6. Maximum Dependable Capacity (Gross MWe)1098.4	
7 Maximum Decendable Capacity (Net MWe): 1065	
. If Channes ( r in Canacity Ratings (Itams Number 1 Through .	St Singa Last Banart Cina Banana

Through 7) Since Last Kep

N/A

60%

 9 Power Level To Which Restricted, If Any (Net MWe): 60%
10. Reasons For Restrictions, If Any: All Automatic Depressurization System Relief Valve Cables are routed through the same cable tray.

	This Month	Yrto-Date	Cumulative
11 Hours In Reporting Period	744	3,647	81,150
12. Number Of Hours Reactor Was Critical	744	3,356.84	53,321.02
13. Reactor Reserve Shutdown Hours	0	290.16	14,190.52
14. Hours Generator On-Line	744	3,312.14	51,804.98
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,456,390	7,943,612	148,088,657
17 Gross Electrical Energy Generated (MWH)	461,560	2,574,720	49,172,008
18. Net Electrical Energy Generated (MWH)	448,374	2,505,370	47,763,973
19. Unit Service Factor	100	90.8	63.8
20. Unit Availability Factor	100	90.8	63.8
21. Unit Canacity Factor (Using MDC Net)	56.6	64.5	55.3
22 Unit Canacity Factor (Using DER Net)	56.6	64.5	55.3
23. Unit Forced Outage Rate	0	6.6	23.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each).

August 1984 (Refuel)

25. If Shut Down At End Of Report Period, Estimated Date of Startup 26. Units In Test Status (Prior to Commercial Operation\* Forecast Achieved INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

(9/77)

### **OPERATING DATA REPORT**

# DOCKET NO. 50-296 DATE 6/1/84 COMPLETED BY Ted Thom TELEPHONE 205/729-0834

# **OPERATING STATUS**

1 Unit Name:Browns Ferry - 3	Notes
2. Reporting Period: May 1984	
3. Licensed Thermal Power (MWt): <u>3293</u> 4. Nameplate Rating (Gross MWe): <u>1152</u> 5. Design Electrical Rating (Net MWe): <u>1065</u>	
6. Maximum Dependable Capacity (Gross MWe): 1098 7. Maximum Dependable Capacity (Net MWe): 1065	3.4
8. If Changes Occur in Capacity Ratings (Items Number 3	3 Through 7) Since Last Report. Give Reasons: N/A
9. Power Level To Which Restricted If Any (Net MWe)	N/A
10. Reasons For Restrictions, If Any:	N/A

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744	3,647	63,575
12. Number Of Hours Reactor Was Critical	0	0	43,087.80
13. Reactor Reserve Shutdown Hours	0	0	3,878.13
14. Hours Generator On-Line	0	0	42,193.71
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	.0	126,307,711
17 Gross Electrical Energy Generated (MWH)	0	0	41,597,620
18. Net Electrical Energy Generated (MWH)	0	0	40,375,256
19. Unit Service Factor	0	0	66.4
20. Unit Availability Factor	0	0	66.4
21. Unit Capacity Factor (Using MDC Net)	0	0	59.6
22 Unit Capacity Factor (Using DER Net)	0	0	59.6
23. Unit Forced Outage Rate	0	0	16.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each).

25. If Shut Down At End Of Report Period, Estimated Date of Startup	August 1984		
26. Units bi Test Status (Prior to Commercial Operation)	Forecast	Achieved	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
CONNERCIAL OPERATION			

DOCKET NO. 50-259 UNIT NAME 1 DATE 6/1/84 COMPLETED BY Ted Thom TELEPHONE 205/729-0834	Cause & Corrective Action to Prevent Recurrence	Derated for "B" string low-pressure heater maintenance.	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) S Exhibit I - Same Swarce
REDUCTIONS	inonoquio) Zobir)		d: aal aal Scram. matic Scram. r (Explain)
POWER	t short		Methos I-Manu 2-Manu 3-Auto 4-Othe
HUTDOWNS AND REPORT MONTH	Licensee Event Report =		e.
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	nonemd (swolf)		n upment Fai ntenance or ueling gulatory Res rator Train ministrative rrational Eri er (Explain
	1 art 21	£4.	E Reaso A-Equ B-Mai D-Reg E-Ope F-Adn F-Adn F-Adn F-Adn B-Reg
	bur-	4/17/84	eduled
	1	281	F Fou S Sch row 77)

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO UNIT NAME DATE COMPLETED BY TELEPHONE

ET NO	50-260
NAME	2
DATE	6/1/84
ED BY	Ted Thom
PHONE	205/729-0834

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# REPORT MONTH May 1984

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1	Date	Type <sup>1</sup>	Duation	Reason 2	Method of Shatting Down Reactor 3	Licenser Event Report #	System Cude <sup>4</sup>	Composition	Cause & Confective Action to Prevent Recurrence
292	5/1/84	s		H					Derated to extend fuel cycle into August 1984.
293	5/10/84	5		Н					Derated to extend fuel cycle and ad- ministrative hold because all Automat Depressurization System relief valve cables are routed through the same cable tray.
1 F: Fo S: Sc P4(77)	rced heduled	Reas A-Eq B-Ma C-Re D-Re F-Ad G-Op H-Op	on: uipment Fa iintenance o fueling gulatory Ro peration Train Immistration perational E her (Explain	nlure (E of Test estriction ning & I e tror (E x n.)	xplain¥ n acense F xa splain¥	manation	Method 1-Manu 2-Manu 3-Auto 4-Other	l: al al Scrain. matic Scrain. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I - Same Source

U					UNIT	SHUTDOWNS AN	D POWER N	REDUCTION	DOCKET NO. UNIT NAME DATE COMPLETED BY TELEPHONE DOCKET NO. 3 <u>6/1/84</u> Ted Thom 205/729-0834	
•	Date	Toper 1 area to a second and the sec		Sy stem Cude <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence				
140	5/1/84	S	744	c	4				End-of-cycle refuel outage continues. (Controlled shutdown 9/7/83)	
1   2     F. Forced   Reason     S. Scheduled   A-Equipmeaute (Exp.     B-Maintenance or Test   C-Refueling     D-Regulatory Restriction   E-Operator Training & La     F-Administrative   G-Operational Error (Exp.     (9/77)   H-Other (Explain)		xplam) n accuse Exan aplain)	510-a14-99	3 Method 1-Manu: 2-Manu: 3-Autor 4-Other	d d Scram. natic Scram. (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161) S Exhibit I - Same Source				

# BROWNS FERRY NUCLEAR PLANT UNIT 0

# CSSC EQUIPMENT

# ELECTRICAL MAINTENANCE SUMMARY

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For the Month of May 1984

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
1984 5/25	Radiation monitoring	0-PMP-90-152A	Replace fuses	None	Improper operation	Sample pump not operable. Low flow alarm	Task force organized to resolve recurring problem. MR 268030
5/28	Standby diesel generators	0-PX-82-00C (NVR)	Replace relay	None	Bad coil	Relay inoperable	Replaced bad relay. MR 267385
5/30	Standby diesel generators	0-RLY-82-00B	Replace relay	None	Bad relay	Relay inoperable	Replaced bad Felay. MR 252792

BROWNS FERRY SUCLEAR PLANT UNIT 1

CSSC EQUIPMENT

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ELECTRICAL MAINTENANCE SUMMARY

Page 3 BF ENSIL 33 Appendix B 9/29/82

For the Month of May 19 84

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Canada Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
<u>1984</u> 5/9	Control rod drive	1-LA-85-85B	Replace printed circuit card	None	Bad card	Alarm	Replaced bad card. MR 265038
5/19	Associated electrical systems	1-RLY-57-16AK2	0 Replace coil	None	Bad coil	Relay inoperable	Replaced bad relay. MR 268034 LER BFR0-50-259/ 84023

· · · ·	Action Taken To Preclude	Replaced bad relay. MR 268302	MB 268300, MS 2683000, MS 268300, MS 2683000, MS 2683000, MS 268300, MS 2683000, MS 2683000000000000000000000000000000000000
2 Park J BF ETSIL A7Pendix 9/29/82	Results of Malfunction	Relay inoperable	Alarm in control room
BEDWISS FERSY KULLAR PRANT UNIT ELECTRICAL MAINTEXANCE SUMMARY For the Month of May 19	Call and of	Bad relay	Unknown
	Effect on Safe Operation of The Reactor	None	None
	Nature of Maintenance	Replace relay	Remounted suoke detector
IPAENT	Component	2-RLY-99- 5AK-5C	2-882-26- 105CT
CSSC EN	Systen	Reactor protection	High pressure fire protection
	Date	5/4	5/5

SE SWES FERRY RUCLEAR PLAST UNIT 3

CSSC EQUIPMENT

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# ELECTRICAL MAINTENANCE SUCCARY

Date 3 BF 11511 33 Aroundia 3 9/29/82

For the Month of May 19 84

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Can	Results of	Action Taken To Preclude
<u>1984</u> 3/28	Control rod drive	3-HCV-85-50-47	Replace switch	None	Bad switch	Switch not operable during performance of EMI 50	Replaced bad switch MR 155983
3/28	Control rod drive	3-HCU-85-46-47	Replace switch	None	Bad switch	Switch not operable during performance of EMI 50	Replaced bad switch MR 155982
4/30	Control rod drive	3-PS-85-34- (46-11)	Replace level switch "O"-ring	None	Bad "O"-ring	Nitrogen leaking	Replaced bad "0"- ring. MR 254144
4/30	Control rod drive	3-1.S-85-34- (50-27)	Replace level switch "O"-ring	None	Bad "O"-ring	Nitrogen leaking around "O"-ring	Replaced bad "0"- ring. MR 254161
5/1	Reactor water recircula- tion	3-FCV-68-33	Replace coil	None	Bad coil	Blowing fuses on Rx MOV Bd 3C	Replaced bad coil. MR 269875
5/7	Standby diesel generator	3-PMP-82-3A	Replace motor	None	Bad motor bear- ings	Pump motor making noises	Replaced bad lube oil pump motor. MR 269873
5/7	CO <sub>2</sub> storage fire protection	3-00-39-00 Cardox unit	Replaced expan- sion valve	None	Bad valve	Cardox unit not cool- ing	Replaced bad expan- sion valve. MR 147355

BROWNS FERRY NUCLEAR PLANT CVIT 3

CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SEMMARY

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For the Month of May 19 84

late	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of	Results of	Action Taken To Preclude
984 5/11	Air condi- tioning	3-AHU-31-0104	Replace contacts	None	Contacts not mak- ing up properly	Air conditioning unit not cooling	Replaced bad contact MR 254308
6/14	Reactor core isolation cooling	3-RLY-72-K12 PNL-25-31	Replace relay	None	Bad relay	Relay not timing correctly	Replaced bad relay, MR 254305
5/18	250VDC System	3-CHGA-248- 03EB	Replace record- ing voltmeter	None	Bad voltmeter	Low reading on volt- meter	Replaced bad volt- meter. MR 217298
5/21	CO2 stor- age & fire protection	3-TA-39-110	Replace cable tray heat detector	None	Bad heat detector (cable found cut in two)	Alarm will not clear	Replaced cable tray heat detector. MR 252885
/23	Standby diesel generator	3-SCO-82-3ED (controls)	Replace speed sensing switch	None	None	None	Replaced old type speed sensing switch with solid-state type(more reliable). MR 254313
5/29	4kV shut- down boards & busses	3-RLY-211- 27R/53CX (HFA relay)	Replace coil	None	None	None	Replace with new type coil. MR 254771

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G a	Action Taken To Preclude	Replaced coil new type. MR 254758	Replaced coil new type. MR 254764
3	Results of	None	None
CLEAR PLANT UNIT	Cluse of	None	None
ELECTRICAL For the Mont	Effect on Safe Operation of The Reactor	None	None
	Nature of Maintenance	Replace coil	Replace coil
162821	Component	3-RLY-211- BBRB-1	3-RLY-211- CASB-2
CSSC EQI	Systen	4kV Shut- down boards & busses	4kV Shut- down boards & busses
	Date	1984	2/30

CSSC EQUIPTENT

BROWNS FERRY NUCLEAR PLANT UNIT 1

MECHANICAL MAINTENANCE SUPPARY

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For the Month of May 1984

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ACTION TAKEN TO PRECLUDE RECURRENCE	MR A-253352 MR A-209481	MR A-206374	MR A-253734	MR A-260373	MR. A-264071	
RESULTS OF MALFUNCTION	e Worn Coupling	Contaminated oil	Servo leaking at seal	N/A	RCIC steam valve packing leaking	
CAUSE OF MALFUNCTION	Normal age and u	Water in oil	Normal age and use	Preventative Maintenance	Normal age and use	
EFFECT ON SAFE OPERATION OF THE REACTOR	g Kone	None	Rone		on kone	
MATURE OF MAINTENANCE	Replace Couplin	Change out oil HPCI turbine	Change out servo	Change out oil in RCLC turbine	Change out carb	•
CONFORENT	1-712-85-0001 n	re 1-T3B-73-0054	e 1-SC-71-0010	1-TRB-71-0009	1-FCV-71-0010	
SYSTEM	Control Rod Drive Syste	High Pressu Coolant Injection (MPCI)	Reactor Cor Isolation Cooling System(SCIC)	Resctor Core 1.01%fion Cooling System(RCIC)	Reactor Core Isolation Cooling System(RCIC)	
DATE	17	12-	5-25	5-25	5-25	

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CSSC EQUIPMENT

DEUMONS FERRY NUCLEAR FLADE UNIT 3

MECHANICAL MAINTENANCE SUMMARY

For the Month of May 19 84

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ACTION TAKEN TO PRECLUDE PECURRENCE	XR A-155333	MR A260300	
RESULTS OF MALFUNCTION	None	None	
CAUSE OF MALFUNCTION	Normal Age and Use	None, Preventative Maintenance	
EFFECT ON SAFE OPERATION OF THE REACTOR	Kone	lione	
MATURE OF MAINTENANCE	Replace flex hose	Clean Cooler after prot clog, reinstall heads & piping	
CONPONENT	3-DC-82-0030	3-DC-82-0030	
SYSTEM	Standby Diesel Genurator System (82)	Standby diesel Generator System	
DATE	5-20	5-21	

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BROWNS FERRY NUCLEAR FLANI UNII

# CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of May 19 84

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE	
5-3	High Pressur Fire Pro- Lection (26)	e 0-HCV-26-0572	Replace Valve	None	Age and raw water	Valve leaking thru	MR A-253354	
5-2	High Pressure Fire Protection	0-SRV-26-0573	Repair Valve	None	Age and raw water	Valve leaking thru	MR A-260362	
							27	
								•

### I. Work Synopsis

Unit 3 cycle 5 outage activities continued through day number 268 by the end of the May report period. Major emphasis areas this month have included P0392 CRD Scram Discharge Header Modification, MSIV Maintenance; P0399 Instrument and Control Bus Modification, and induction heat stress improvement (IHSI) of selected weld joints, as well as work on the refuel floor and in the torus. The following is a summary of the major events during this work period.

### A. Critical Path Activities

- PO392 CRD Scram Discharge Header Modification Piping and valve tagging was completed on May 5, 1984. Hydrostatic test of the headers was completed on May 8, 1984. Platform work is continuing.
- 2. P0392 CRD Scram Discharge Header Hanger/Support Modification Of 119 hangers, 82 have been completed with 17 having greater than 75-percent of their work complete. Many of the 37 incomplete hangers are awaiting FCR resolutions. Detailed breakdown is as follows:
  - a. Of twelve locations requiring inspection of existing hangers, 11 are complete.
  - b. Of 47 locations requiring removal only, 40 are complete.
  - of 18 locations requiring removal and installation of new hangers, 17 are complete.
  - d. Of 38 locations requiring new hangers, 12 are complete.
  - e. Of four locations requiring a modification to an existing hanger, two are complete.

- A. <u>Critical Path Activities</u> (Continued)
  - PO392 (Instrumentation) This work is continuing with pulling and terminating of cable.
  - PO399 Instrument and Control Bus Modification This work is continuing with the running of conduit and pulling of cable.
  - MSIV Maintenance This work is continuing with reassembly being started on several of the valves. PO614 Stem Modification, was reported complete on May 20.
- B. <u>Refuel Floor</u>

PO334 Changing Air Compressor on Platform was completed. PO027 Removal of Old Racks in Preparation for High Density Fuel Storage Rack Installation was completed on May 18. Fuel pool shuffle work began on May 22 and 351 of 930 moves had been completed by month end (one shift per day). Emphasis has been started on scheduling the repair of the jet pump nozzle which was found cracked during inspection. This work would involve a possible drain down of the reactor vessel.

# C. Balance of Plant

Work continued on "C" Residual Heat Removal (RHR) heat exchanger. Eddy current testing revealed seven tubes to require plugging. The tubes were plugged and heat exchanger closed by the end of the month. Final torque and installation of drain piping are still needed to complete work. The condenser work was reported complete on May 3.

# D. <u>Turbine</u>

The generator air test was completed on May 21. Replacement of the worn wheels on unit 3 turbine building crane was begun during May. Two of the four wheels have been completed.

#### E. Induction Heat Stress Improvement (IHSI)

The IHSI process was completed during May. The final recirculation weld was completed on May 15. The final RHR weld was treated on May 17. The final RWCU weld was completed on May 25 and the preparations for the GE contractor to move out began.

# F. Other Mechanical Work

- Diesel Generators L1970/P0709 EECW piping and valve changeout was completed on "3D" and "3C" diesels. Work began on "3B" diesel on May 30 and is continuing. Work on "3A" scheduled to start following the completion of "3B."
- 2. Hydraulic Snubbers This work continued during the month.
- 3. MSRV's Installation of valves and solenoids was completed. PO612, Installation of 1/2-inch SS flex line, is needed for completion. Six of 13 hangers have been designed for PO612 while material is being obtained for fabrication.
- P0689 and P0691 64 series valves. This work is continuing and awaiting delivery of O-rings.
- P0688 and P0690 Modify valves 76-18/19. This work continued during the month. Both valves were electrically hooked up May 25, 1984.

- F. Other Mechanical Work (Continued)
  - PO695 Modify valves 84-8A, 8B, 8C, and 8D. This work continued during the month.
  - L2079 H<sub>2</sub>O<sub>2</sub> Modification. This work began during May. Work was temporarily delayed due to need for 1/2 inch pipe caps. These caps were received May 30 and work resumed.
  - 8. PO684 Drywell Torus Vacuum Breakers This work is continuing.
  - 9. P0569 Replace RPV Head Vent Valves Work continued during the month.
  - Repaired RHR suction valves 74-85/88. Both valves were stuck in open position.
  - 11. Valves 77-2A/2B and 77-15A/15B passed Local Leak Rate Test (LLRT).
  - 12. Valves 71-32/592 passed LLRT.
  - 13. Valves 73-24/609 passed LLRT.
  - Replaced seals on unit 2 "C" RHR pump seal cooler heat exchanger on May 3.
  - Repaired leakage on unit 2 "A" RHR heat exchanger. A Limiting Condition of Operation (LCO) began on May 20
  - PO730 Head Spray Piping Removal Hold order established to begin piping removal on May 28.
- G. Electrical/Instrumentation
  - Diesel Generators Completed modifications P0185 Rewiring of Diesel Protective Circuit, P0275 Degraded Voltage Relay Replacements, and P0585 Speed Sensing Device on "3D" and "3C" diesels. Work has started on "3B."

- G. <u>Electrical/Instrumentation</u> (Continued)
  - 2. EMI 71 Motor Maintenance Core spray "3B" and "3C" motors were completed on May 2. Closed Cooling Water (CCW) pump motors "3A" and "3B" were completed on May 5. Raw Cooling Water (RCW) pump motors "3B" and "3C" were completed May 4. RHR service water pump motor "3A" was completed on May 15. High Temperature Circulation pump motors "A" and "B" were completed on May 16. Five motors remain to be tested for this outage.
  - FMI 7 Electrical Board Maintenance Five breakers remain this outage.
  - PO415 Reactor Pressure Vessel Feedwater Nozzles This work is continuing.
  - PO322/323 Torus Level Transmitters Work is continuing. Awaiting receipt of level transmitters. Expected date of delivery is June 15.
  - 6. PO631 RHRSW Tunnel Radiation Monitors Work plan is in review cycle.
  - P0533 Install Torus Temperature Devices Work continued but is delayed pending the completion of P0126, installation of inverters, for power.
  - 8. Inspected EHC system cables for damage due to fluid spills.
  - 9. TIP Indexer Maintenance Sealing started on May 13.
  - PO284 Acoustic Monitors This modification began during the month and is continuing.
  - LLRT was completed on the electrical penetrations of the TIP ball and check valves

#### H. Torus

Torus work was as follows:

- Torus attached piping support installation continued. By the end of the month, 356 of 456 supports were complete.
- 2. Repair of valve 74-722 was completed on May 6.
- 3. Touch-up painting of torus was completed.
- 4. Torus fill was completed on May 26.
- NRC 79-02 hanger inspection started during the month with 95 of 362 complete.

## I. Planning and Scheduling

Planning and scheduling efforts during this report period emphasized maintaining unit 3 cycle 5 network of activities while closely monitoring critical path activities. Unit 2 cycle 5 planning efforts are in progress. However, lack of confirmation on modifications to be worked coupled with lack of information on the scope of work involved with selected modifications is hampering this iffort.

### J. Administrative

The overtime percentage for the month of April was 17-percent with 115,829 straight time hours and 23,635 overtime hours. As of April 30, 1984. yeartc, date overtime percentage was 20-percent, with 917,086 straight time hours and 227,391 overtime hours. The overall goal of the overtime percentage is 17-percent.

# J. Administrative (Continued)

The Outage & Maintenance budget for April was \$2,576,410 and the expenditures were \$2,494.223 with year-to-date budget being \$19,109,384 and actual year-to-date expenditures being \$19,169,934.

The capital budget was \$3,194,000 and the expenditures were \$6,446,281 with year-to-date budget being \$24.662,900 and actual year-to-date expenditures being \$25.727,507. Overall budget was \$5,770,510 and the overall expenditures were \$8,940,504 with year-to-date budget being \$43,772,284 and actual year-to-date expenditures being \$44.897,442.