

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 2 2 0	PAGE (3) 1 OF 0 2
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TITLE (4)
High Pressure Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	2 1	8 4	8 4	0 0	7 0	0 6	2 1	8 4			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (8) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)					
	20.406(a)(1)(i)	80.36(a)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)					
	20.406(a)(1)(ii)	80.36(a)(2)	<input type="checkbox"/>	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.406(a)(1)(iii)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)						
	20.406(a)(1)(iv)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)						
20.406(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Robert Randall, Supervisor, Technical Services	TELEPHONE NUMBER 3 1 5 3 4 9 - 2 4 4 5
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

During a refueling outage, shortly after control rod scram time testing was performed, two high pressure reactor scrams occurred, within 9 minutes of each other. The reactor vessel was under post-hydrostatic testing conditions with the reactor vessel water solid. The reactor vessel pressure was being manually controlled by a "feed and bleed" process. Pressure oscillations due to control rod scram time testing performed just prior to the initial event compounded the difficulty in maintaining the reactor vessel pressure. As a direct result of these conditions, high pressure fluctuations occurred, which initiated automatic high pressure reactor scrams. Procedural changes will be incorporated so as to prevent this type of event from occurring in the future.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Nine Mile Point Unit #1	DOCKET NUMBER (2) e 5 0 0 0 2 2 0	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 0 7	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TEXT

On May 21, 1984, during a refueling outage, at about 6:11 am, control rod scram time testing was in progress. The reactor vessel was under post-hydrostatic testing conditions with the vessel water solid (ie-no gas volume). The reactor was in cold shutdown, the mode switch was set to "refuel", the reactor pressure was approximately 1040 psig, and all control rods were fully inserted. Control rod scram time testing performed just prior to the initial event compounded the difficulty in manually maintaining the reactor vessel pressure. As a direct result of all of these conditions, a short duration high pressure fluctuation occurred, which initiated an automatic high reactor pressure scram. The scram signal was reset, and approximately 9 minutes after the initial event occurred, a second high reactor pressure scram occurred. This scram is attributed to the same cause as the initial event.

ASSESSMENT OF SAFETY CONSEQUENCES

There are no potential safety consequences arising out of these events because: 1) The reactor was in shutdown and subcritical; 2) the mode switch was set to "refuel"; 3) all control rods were fully inserted at the time of each event; 4) the short duration pressure fluctuations are experienced only when the vessel is in a water solid condition, which only exists under hydrostatic testing conditions; and 5) per Generic Letter 83-28, reactor startup (but not scram time testing) was administratively prohibited until the scram was fully evaluated.

CORRECTIVE ACTION

The Reactor Protection System (RPS) reactor pressure transmitters and trip units were checked and found to be within their setpoint tolerances of 1068 psig ± 12 psig. Procedural changes will be incorporated so as to prevent this type of event from occurring in the future.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

June 21, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

RE: Docket No. 50-220
LER 84-07

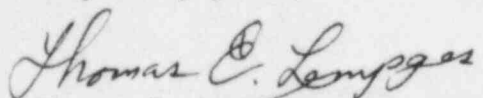
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following
Licensee Event Report:

LER 84-07 Which is being submitted in accordance with 10
CFR 50.73 (a)(2)(iv), "Any event or condition that
resulted in manual or automatic actuation of any
Engineered Safety Feature (ESF), including the
Reactor Protection System (RPS)."

A 10 CFR 50.72 report was made at 0655 hrs on May 21, 1984. This
report was completed in the format designated in NUREG-1022, dated
September 1983.

Very truly yours,



Thomas E. Lempsges
Vice President
Nuclear Generation

TEL/lo
Attachments
cc: Dr. Thomas E. Murley
Regional Administrator

IER2
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