

MONTHLY OPERATIONS REPORT

MAY 1984

Throughout the report period, the Oyster Creek Station remained shutdown for the Maintenance and Refueling Outage.

Channeling of the reactor fuel was completed on May 1, 1984 with refueling of the reactor restarting late the same day. Refueling continued until May 13, 1984 at which time a black and white pattern was reached and refueling was stopped to install channel clips on 77 new fuel assemblies. Refueling of the reactor recommenced and was completed on May 22, 1984 except for those fuel assemblies (62) which were not loaded into the core to support SRM/IRM dry tube replacement. Various delays in the refueling process were experienced during the report period. They consisted of: nuclear instrumentation problems (SRM23), failure of "B" Control Rod Drive (CRD) pump (oil cooler leak), IRM range switch relay failure, rod block problems with the bridge track switch and CRD hydraulic control units; and problems with the Refuel Bridge communication system. Some time was also spent removing debris from the reactor. At the end of the report period, fuel moves were in progress to support LPRM replacement. There is a total of five (5) LPRMs left to be replaced.

At the end of the report period, preparation was in progress to replace all IRM/SRM dry tubes. This task has been contracted out to General Electric Company. Work is expected to start the first week in June.

On May 28, Emergency Diesel Generator No. 2 was declared inoperable after it failed to start during routine testing. The problem was traced to the actuator on the Diesel Generator governor. The governor replacement was in progress at the end of the report period. Standby Gas Treatment System No. 2 is also considered out of service due to Diesel Generator No. 2 being inoperable.

CRD pump "A" was returned to an operable status on May 16, 1984 after an overhaul and corrective maintenance was performed on it due to an oil leak and failed thrust bearing. Corrective maintenance on "B" CRD pump was also performed due to an oil cooler leak. Maintenance on "B" CRD pump caused some delays in the refueling activities as mentioned above since "A" pump was already out of service.

Fire diesel pump No. 2 was out of service from May 15, 1984 to May 21, 1984 to repair a leak in its cooling system radiator. An oil leak on fire diesel pump No. 1 was also repaired during the report period.

Air compressor No. 1 was out of service three times during the report period for oil leaks, interstage relief valve problems and instrument calibrations.

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"A" condensate pump was returned to an operable status and realignment of the condensate pump suction header was completed. Replacement of the expansion joints on the suction of the condensate pumps is currently in progress. "B" pump has been completed while work on "A" and "C" pumps continues. The plant is experiencing problems isolating the "A" and "C" pumps.

Alignment and calibration of the turbine control system continued during the report period. Approximately 2000 gallons of lube oil was added to the turbine lube oil tank. No. 1 and 3 turbine oil lift pumps were replaced.

The overhaul of 1-3 dilution pump was completed. At the end of the report period, a 24-hour test run of the pump remained to be performed. This test run was started on June 1 and June 2, 1984 at which time the pump tripped on low seal water flow. The trip problem is being investigated.

No leaks could be found in either of the fuel pool heat exchangers as testing continued to identify the source of the suspected tube leak. Both heat exchangers were isolated and inspected for leaks a number of times. The system is presently being monitored while the Maintenance and Operations Departments discuss other possible methods that may be used to identify the source of the leak.

Service water pump No. 2 discharge bell was weld repaired, and upon placing the pump back in service, a small leak was noted. New discharge bells arrived on site for both pumps at the end of the report period. The new pump bells will be installed in the near future.

Weld repair of Reactor Building Closed Cooling Water (RBCCW) heat exchanger 1-1 was in progress at the end of the report period.

Installation of the nitrogen layup system for the feedwater heaters was completed.

Cask shipments of CRD blades continued during the report period. On June 3, 1984, the third cask left the site.

The NRC annual plant emergency drill was held on May 10, 1984. The subsequent critique identified a number of items that will be addressed. Overall, performance for the drill was satisfactory.

Torus paint heat curing started on May 17, 1984 but had to be secured on May 19 due to the inability of the heat cure system to maintain proper differential temperatures between the torus shell and columns. Additional heat trace equipment was installed and heat curing of the torus restarted on June 1, 1984.

Testing of the plant's electrical distribution system in the Turbine Building continued during the report period. A defined repair scope is in the process of being developed by Plant Engineering.

New "HFA" relays are expected to arrive on site some time in June. The existing relays which were replaced earlier in the outage are suspect due to an oil leaching problem.

Repairs and analysis of the isolation condenser piping continued during the report period.

Repairs to the drywell cooling system continued during the report period. New cages and fan shafts need to be installed. The fans were rebuilt earlier in the outage but their condition has deteriorated to the extent that extensive repairs are again required. Technical Functions is investigating the reason for the premature degradation.

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The following Licensee Event Reports were submitted during May 1984:

Licensee Event Report No. 50-219/84-004

On April 21, 1984, a contractor found that both the inner and outer doors of the 23'6" elevation airlock of the reactor building were open at the same time. This is in violation of the Technical Specifications, which require that at least one airlock door be closed at all times to maintain secondary containment integrity. The doors were closed immediately after they were found open to restore secondary containment integrity. The apparent cause of occurrence is attributed to either improper or unauthorized use of the electro-mechanical interlock override switch or to the simultaneous opening of both doors which results in both doors being held open. Corrective actions include an increased emphasis, through training and procedures, on the importance of maintaining at least one airlock door closed at all times.

Licensee Event Report No. 50-219/84-005

The shell side of both Isolation Condensers (NE01-A, NE01-B) was hydroblasted and coated to protect against corrosion. As part of post-maintenance testing, the tube side of the condensers was hydrostatically tested to verify tube integrity. On March 22, 1984, during the hydro test on Isolation Condenser NE01-A, a leak was noticed near an insulated elbow on the condensate piping, downstream of the condenser. The insulation was removed and the weld was examined ultrasonically, which pinpointed the existence of a through wall crack approximately 5 inches long. The indications are that the failure may be attributed to Intergranular Stress Corrosion Cracking.

The entire Isolation Condenser System is being inspected to determine the integrity of the remaining weld joints. Only then can a program be developed to formulate the corrective action required.

Licensing Event Report 50-219/84-006

As a result of blast clearing Torus structure components during the 1983 refueling/maintenance outage, areas of corrosion pitting were uncovered, evaluated and repaired where necessary. In addition, missing fillet welds on two (2) of twenty (20) ring girders were found, as were missing fillet welds on ring girder flange to web reinforcing plates. All missing welds were repaired. The determination of the repairs required, and the performance of the repairs, were in compliance with ASME Code Requirements, and all other applicable requirements including Mark I Containment Program requirements. This report was considered a voluntary report.

## Licensee Event Report No. 50-219/84-007

On April 2, 1984, Diesel Generator No. 1 (DG-1) was declared inoperable as a result of a failure to fast start during a monthly surveillance. Since DG-1 is the emergency power supply for Standby Gas Treatment System No. 1 (SGTS-1), SGTS-1 was also declared inoperable. Technical Specifications require that the redundant SGTS train be demonstrated operable within two (2) hours. SGTS-2 was not tested, however, until the completion of a ten (10) hour wait period that was necessary to prevent degradation of the filter trains from the fumes from painting work in the torus. Torus painting was stopped when the requirement to test SGTS-2 was established, and SGTS-2 was tested after the ten (10) hour wait period. A change to the Technical Specifications will be investigated to determine whether the restrictive requirements regarding normal and emergency power supplies (in the Shutdown and Refuel modes) can be eased or clarified. The procedural requirement to wait ten (10) hours prior to operating a SGTS train for testing, after termination of painting activities, will be changed when a test is required to comply with an LCO.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May 1984

DOCKET NO. 50-219  
 UNIT NAME Oyster Creek  
 DATE 6-4-84  
 COMPLETED BY R. Baran  
 TELEPHONE 971-4640

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	11400	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

OPERATING DATA REPORT  
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: May, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe):  $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	744.0	3648.0	126576.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0.0	0.0	136224729
18. GROSS ELEC ENER (MWH)	0.0	0.0	46056905
19. NET ELEC ENER (MWH)	-2116	-9732	44275951
20. UT SERVICE FACTOR	0.0	0.0	65.3
21. UT AVAIL FACTOR	0.0	0.0	65.3
22. UT CAP FACTOR (MDC NET)	0.0	-0.4	56.4
23. UT CAP FACTOR (DER NET)	0.0	-0.4	53.8
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	8/07/84		

AVERAGE DAILY POWER LEVEL  
NET MWe

DOCKET # . . . . . 50-219  
UNIT . . . . . Oyster Creek #1  
REPORT DATE . . . . . JUNE 04, 1984  
COMPILED BY . . . . . DONALD V. NOTIGAN  
TELEPHONE # . . . . . 609-971-4695

MONTH MAY, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0
		31.	0



REFUELING INFORMATION - May, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: 8/07/84

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

NONE

Scheduled date(s) for submitting proposed licensing action and supporting information:

1. Amendment #7 to NEDO-24195: G.E. Reload Analysis incorporating LOCA analysis. (5/30/84)
2. Technical Specification Change Request 119 for Scram Discharge Instrument Volume System Modification. (5/15/84)

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 492  
(b) in the spent fuel storage pool = 1049

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800                      Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.



GPU Nuclear Corporation  
Post Office Box 388  
Route 9 South  
Forked River, New Jersey 08731-0388  
609 971-4000  
Writer's Direct Dial Number:

June 14, 1984

Director  
Office of Management Information  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:dsm  
Enclosures

cc: Director (10)  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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