NRC For (9-63)	m 386									LIC	ENS	EE	EV	EN	T RE	PORT	(LE	R)			U	I.S. N	APT	ROV	VED (		1	1 CON	-	SION
FACILITY	Y NAME	(1)		_		_	-	_	-			-		_					100	CKET	TNU	MBE	R (2)	)	_		_	P	AGE	(3)
[2] (1] [2] (1] (1] (1] (1] (1] (2] (2] (2] (3] (4] (4] (4] (4] (4] (4] (4] (4] (4] (4									-				100	101213 7				10	Fh	12										
TITLE (4																			1	1	1	1			<u> </u>	) V		. 10	· D	12
evi	ENT DAT	(a)	cdo	X						Valve			ORTDA					OTH		C11 1	7100		21.10		781					
MONTH DAY YEAR				YEAR SEQUENTIAL						REVISION				_	EAR	FACILITY				A FACILITIES INVOLV				OOCKET NUMBER(S)						
						NUMBER				NUMBER	MUNTE	+	DAY	+	TEAN	N/A			0  5   0   0											
0   5	3   0	8	4	8	4 -	0	0	8	-	0 0	0 6	,	2  5	8	14								0	1	510	0 10	0 1	01	1	1
	RATING DE (9)		N	THIS	REPOR	TISE	UBM	ITTED	PUR	SUANT	TO THE	RE	QUIRE	MENT	8 OF 10	CFR §: /	Check	ne or m	ore of t	the fo	i/owi	ing) (	11)	_			_			
POWER LEVEL 0 1010			0	20.402(b) 20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)						20.406(e) 50.36(e)(1) 50.36(e)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii) ICENSEE CONTACT FOR THIS LER						50.73(a)(2)(v) 50.73(a)(2)(vii) 50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B) 50.73(a)(2)(xiii)(B)					73.71(e) 73.71(e) OTHER (Specify in Abstract below and in Text, NRC Form .166.A)									
NAME													CHIAC		11110	EEN (12)				T		-	TE	LEF	HON	E NUI	MBE	R	_	
		s.	Me	rri	itt												(X	-421	)	-	EA C			14	2	1.	. ,	219	9,2	2 10
						co	MPL	ETE O	NE L	INE FOR	EACH (	COA	MPONE	NT FA	MLURE	DESCRIBI	ED IN	HIS RE	PORT	13)	-	_	-	_			_			
CAUSE	SYSTEM	co	ОМРО	NENT		MANUFAC- TURER			REPORTABLE TO NPROS						CAUSE	SYSTEM	co	COMPONENT		MANUFAC TURER					ORT.					
D			_			L										1				1										
	1			1												1	1	1		1	1	1								
	(If yes, c				THE STATE OF	MISSI	0 N D	ATE)		REPORT	T	X	NO NO							EXPECTED SUBMISSION DATE (15)				MONT	*	DAY	*	EAR		

During an NRC audit of Dresden Station's compliance with the Fire Protection Technical Specifications it was noted that the Cardox System master valve was not being tested in the automatic mode. The test as written only tested the valve manually. The valve was immediately taken out of service per Technical Specifications and a test of the automatic function was conducted. Following the test the valve was returned to service. The surveillance procedure will be revised to test the valve in both manual and automatic modes.

8407020330 840625 PDR ADDCK 05000237 PDR

理门

NRC Form 366A

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104

	EXPINES 8/31/85												
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)										
		YEAR SEQUENTIAL REVISION NUMBER NUMBER											
Dresden Nuclear Power Station	0  5  0  0  0  2  3  7	84-0p18-p100	12 OF 0   2										

EXT (If more space is required, use additional NAC Form 366A's) (17)

During routine NRC inspection of Dresden's Fire Protection Program, it was discovered that no documentation existed to verify that the Cardox system electro-mechanical master pilot valve was operable in the automatic mode. The Cardox System was declared inoperable immediately. An hourly fire inspection was established per Tech Spec 3.12.D.4, with backup fire suppression equipment in unprotected areas.

The electro-mechanical master pilot valve controls the position of each selector valve for U2, U3 and U2/3 (selector valves control the flow of  $\rm CO_2$  into the diesel generator rooms). In Procedure DFPP 4145-1, revision 1, Cardox System Semi-Annual Maintenance Test, the master pilot valve is verified for manual actuation only. Special Procedure 84-5-35 was written, on-site reviewed, and performed to verify automatic actuation. The Cardox System was declared operable at 1630 hours on 5/30/84.

Automatic actuation of the master pilot valve will be added to DFPP 4145-1.

Safety significance was minimal due to the fact that the master pilot valve was verified operable per design in the automatic mode. In addition, each diesel generator day tank was protected by its wet pipe sprinkler system.

This is the first occurrence of this kind at Dresden Station.

June 25, 1984

DJS Ltr #84-613

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Licensee Event Report #84-008-0, Docket #050-237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

D.J. Scott

Station Superintendent Dresden Nuclear Power Station

DJS/kj1

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
 File/NRC
 File/Numerical

IES!