LICENSEE EVENT REPORT (LER)						U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMS NO. 3180-010 N EXPIRES - 6/31/83					
FACILITY NAME								OCKET NUMBER	(2)	FAGE	
Nine Mile Point Unit #1						0 5 0 0 0 2 2 0 1 OFD					
	r Scram du	e to Low	Instru	ument Air	Head	er Pr	essure				
EVENT DAT	E (6)	LER NUMBER IS	u	REPORT DAT	E (7)		OTHER	ACILITIES INVOL	VED (8)		
MONTH DAY	YEAR YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	REVISION HOUTH DAY THE SACILITY NA			MES DOCKET NUMBERISI				
									0 5 0 0	1011	
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MODE (8)	N 20.40			20.406(e)		X	\$0,73(a)(2)(iv)		73,71(b)		
LEVEL		6(a)(1)(8) 6(a)(1)(8)	-	80,36(s)(1)			60.73(a)(2)(v)		72.71(4)		
1101 [()		0 (a)(1)(M)		80,36(c)(2) 80,73(c)(2)(ii)		H	50.73(a)(2)(vii) 50.73(a)(2)(viii)(A		DETHER ISM	TORL MAC P	
		8 to)(1)(tr)		80.73m)(2)(0)		H	80,73(a)(2)(viii)(8				
	30.40	6 (a) (1) (v)		80.73(a)(2)(iii)			80.73(a)(2)(x)				
NAME			L	CENSEE CONTACT	FOR THIS	LER (12)			TELEPHONE NUM	**	
								AREA CODE	TELEPHONE NOM		
Robert F	andall - S	Superviso	or, Tec	hnical Se	rvic	es		31115	314191-	12 14 14	
		COMPLETE O	NE LINE FOR	EACH COMPONENT	FAILURE	DESCRIBE	IN THIS REPORT	(13)			
CAUSE SYSTEM	COMPONENT	MANUFAC TURER	TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO HPADS	•	
1	111	111					111	111			
		SUPPLEMEN	TAL REPORT	EXPECTED (14)	1				MONTH	DAY Y	
								BUBMISSIC DATE 116	ON		
	ompiou EXPECTED SU			X NO				DAIL 111			
During	a refuelin	g perfor	med. 1	clic surve Due to an e test.	oper	ator	error, a	blockin	g valve	was	

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NAC Farm MAA (9 8)1	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION								APPROVED ONE NO 3150-0104 Expires 8/31/46					
FACILITY NAME (1)		DOCKET NUMBER (2)	LEA NUMBER (6)					PAGE (3)						
			***		STOUTATIAL SUMBIA				T	T				
Nine Mile Po	oint Unit #1	0 5 0 0 0 2 2 0	8 14	_	01018	-	010	0 12	OF	0	12			

During a refueling outage on May 24, 1984, a cyclic surveillance flow test of the containment spray nozzles using instrument air was being performed. Due to an operator error, a blocking valve between the containment spray system air test tank and the instrument air system was not closed, which was a violation of the surveillance test procedure. When the test was performed, this open valve caused the instrument air pressure to drop, causing the reactor scram valves to open. This started filling the scram discharge volume, which initiated an automatic scram signal.

ASSESSMENT OF SAFETY CONSEQUENCES

The loss of instrument air throughout the plant resulting from the blocking valve being open could have caused any equipment which is supported by this system to not function properly. However, the potential safety consequences arising out of this event are negligible because: 1) the plant is designed to be fail safe on loss of instrument air: 2) the reactor was in shutdown and subcritical; 3) the reactor mode switch was in "shutdown"; 4) the control rods were all fully inserted at the time of the event; and 5) a manual reactor scram is required when the instrument air pressure drops below 75 psig; therefore, a scram would have ordinarily occurred before the reactor scram valves opened, had the rate of pressure change in this incident been low enough to allow time for a manual scram to take place.

CORRECTIVE ACTION

The scram was reset, the scram discharge volume was drained, the blocking valve between the containment spray air test tank and the instrument air system was closed, instrument air pressure was returned to normal, and the surveillance test was then resumed and successfully completed. The operator involved with the error was reinstructed by the Operations Supervisor on the importance of strict compliance with all operating procedures.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA MOHAWK

300 ERIE BOULEVARD, WEST SYRACUSE, N. Y. 13202

June 25, 1984

United States Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

RE: Docket No. 50-220 LER 84-08

Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following Licensee Event Report:

LER 84-08

Which is being submitted in accordance with 10 CFR 50.73 (a) (2) (iv), "Any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS)."

A 10 CFR 50.72 report was made at 1937 hrs. on 5/24/84. This report was completed in the format designated in NUREG-1022, dated September 1983.

Very truly yours,

Thomas E. Lempges Vice President Nuclear Generation

TEL/cma Attachments (3 copies)

cc: Dr. Thomas E. Murley Regional Administrator

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