

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7	PAGE (3) 1 OF 0 2
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TITLE (4)  
Number of Reactor Coolant Pumps Required in Mode 3

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	3	1 8 4 8 4		0 3 8	0 0	0	6	2 6 8 4	Sequoyah, Unit 2		0 5 0 0 0 3 2 8
									0 5 0 0 0		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 20.406(b)	<input type="checkbox"/> 20.406(c)(1)	<input type="checkbox"/> 20.406(c)(2)	<input type="checkbox"/> 20.406(c)(2)(i)	<input checked="" type="checkbox"/> 20.406(c)(2)(ii)	<input type="checkbox"/> 20.406(c)(2)(iii)	<input type="checkbox"/> 20.406(c)(2)(iv)	<input type="checkbox"/> 20.406(c)(2)(v)	<input type="checkbox"/> 20.406(c)(2)(vi)	<input type="checkbox"/> 20.406(c)(2)(vii)(A)	<input type="checkbox"/> 20.406(c)(2)(vii)(B)	<input type="checkbox"/> 20.406(c)(2)(viii)	<input type="checkbox"/> 20.406(c)(2)(ix)	<input type="checkbox"/> 20.406(c)(2)(x)	<input type="checkbox"/> 73.71(b)	<input type="checkbox"/> 73.71(c)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
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LICENSEE CONTACT FOR THIS LER (12)

NAME Glenn B. Kirk, Compliance Section Engineer	TELEPHONE NUMBER 6 1 5 8 7 0 - 6 1 4 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Sequoyah technical specifications require two reactor coolant pumps operable with one pump in operation for mode 3 operation. A Westinghouse review has determined that with only one reactor coolant pump operating, the DNB design basis may not be met during a bank withdrawal from subcritical accident.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Sequoyah, Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 2 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 3 8	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Technical specification 3.4.1.2 requires two reactor coolant pumps to be operable with one pump in operation for mode 3 operation. Mode 3 is defined by the technical specifications as reactor coolant system temperature between 350 degrees F and no-load temperature of 547 degrees F with the reactor subcritical as required by the shutdown margin technical specification 3.1.1.1.

At 1600 EST on 05/31/84, Westinghouse notified Sequoyah that consistency between the safety analysis and technical specifications as required by 10 CFR 50.36 may not exist. Mode 3 operation is bounded by the analysis performed for hot zero power. These analyses were performed assuming that either two or all of the reactor coolant pumps are operating. The limiting accidents for hot zero power are main steam line break, rod ejection, and bank withdrawal from subcritical.

Westinghouse has reviewed these accidents under the reduced flow conditions of one pump and determined for the steam line break and rod ejection events that the inconsistency between the safety analysis and the technical specifications will not impact the conclusions presented in the safety analysis. For the bank withdrawal from subcritical accident, Westinghouse calculations showed that the DNB design basis for the condition II event may not be met when only one reactor coolant pump is in operation. Based on this evaluation, the margin for safety as defined in the basis for technical specification 3.4.1.2 is reduced and the condition may be an unreviewed safety question according to 10 CFR 50.59.

Westinghouse has stated that on a best estimate basis, DNB design basis can be met since the licensing basis analysis includes conservatisms (such as high reactivity insertion rates) which when removed show that the DNBR is above limits. Therefore, no significant safety hazard exists.

Upon notification of this condition, Sequoyah implemented administrative controls to require either (1) two reactor coolant pumps operating in mode 3, or (2) one reactor coolant pump operating with all rods on the bottom with all power removed from the rods when in mode 3. The administrative controls will remain in effect until a technical specification change can be submitted to and approved by the NRC.

There was no effect on public health or safety. At the time of the notice, unit 1 was in mode 1 (550 degrees F, 2235 psig) at 10% reactor power, and unit 2 was in mode 1 (578 degrees F, 2235 psig) at 100% reactor power. There have been no previous occurrences.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant  
Post Office Box 2000  
Soddy Daisy, Tennessee 37379

June 26, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

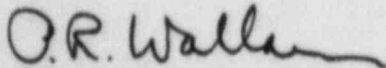
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.  
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT  
SQRO-50-327/84038

The enclosed licensee event report provides details concerning notification from Westinghouse that technical specification 3.4.1.2 may be incorrect. This event is reported in accordance with 10 CFR 50.72, paragraph a.2.ii.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace  
Plant Manager

Enclosure  
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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