

NRC FORM 366
(12-81)
10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011

CONTROL BLOCK: (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(2) (3) (4) (5)

CON'T
 REPORT SOURCE: (6) (7) (8) (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
 On 8/1/80, while placing the plant in cold shutdown, a RCIC inboard isolation valve failed to close on a reactor low pressure signal. The motor overload annunciator activated and the breaker tripped. Reactor pressure was below limits set in Technical Specification 3.5.D.1. This event caused no threat to the public health and safety.

SYSTEM CODE: (11) CAUSE CODE: (12) CAUSE SUBCODE: (13) COMPONENT CODE: (14) COMP. SUBCODE: (15) VALVE SUBCODE: (16)

(17) LER/RO REPORT NUMBER: (18) ACTION TAKEN: (19) FUTURE ACTION: (20) EFFECT ON PLANT: (21) SHUTDOWN METHOD: (22) HOURS: (23) ATTACHMENT SUBMITTED: (24) NPD-4 FORM SUB.: (25) PRIME COMP. SUPPLIER: (26) COMPONENT MANUFACTURER: (27) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

The valve motor was found to be burned out and contactor seized. To preclude recurrence, the valve and operator were replaced and back-seat procedures were revised. Probable cause is attributed to the valve being on backseat during cool-down causing binding of the valve. A review of records indicates no subsequent occurrences of a similar nature.

FACILITY STATUS: (28) % POWER: (29) OTHER STATUS: N/A (30) METHOD OF DISCOVERY: (31) DISCOVERY DESCRIPTION: Operational Event (32)

ACTIVITY CONTENT RELEASED OF RELEASE: (33) AMOUNT OF ACTIVITY: N/A (35) LOCATION OF RELEASE: N/A (36)

PERSONNEL EXPOSURES NUMBER: (37) TYPE: (38) DESCRIPTION: N/A (39)

PERSONNEL INJURIES NUMBER: (40) DESCRIPTION: (41)

8407020297 840613
PDR ADOCK 05000293
S PDR

LOSS OF OR DAMAGE TO FACILITY TYPE: (42) DESCRIPTION: (43)
 PUBLICITY ISSUED DESCRIPTION: N/A (44) (45)

1/1 IE22

ATTACHMENT TO LER 80-044/03X-1

On 8/1/80, RCIC inboard isolation valve MCV 1301-16 failed to close on a reactor low pressure signal, while the plant was being placed in cold shut-down. The Motor overload annunciator came in and the breaker tripped. Reactor pressure was below limits set in Technical Specification 3.5.D.1.

Upon investigation, the valve motor was found to be burned out and the contactor seized. The motor was re-wound and installed, the contactor overload and heater replaced, and a new torque switch installed.

The most probable cause is attributed to the valve having been electrically backseated during cool-down, thus causing binding of the valve and eventual burnout of the motor, since there is no overload protection under isolation signal conditions.

Maintenance records for the valve were examined revealing a history of motor problems requiring repair and replacement on three previous occasions. Other problems which may have contributed to the motor failure include backseating of the valve on numerous occasions to reduce excessive steam leakage causing potential stem alignment problems, plus seal welding due to excessive steam leakage between the body and the bonnet. The valve body was also found to be out of round in the vicinity of the seal ring.

To preclude a recurrence of this type of event resulting from the problems mentioned above, the valve and valve operator were removed and replaced on 3/6/81. Also, a new procedure has been prepared and approved which provides instructions on electrically backseating motor-operated valves remotely when the valves are not accessible during plant operation.

A review of the records indicated that no subsequent events of a similar nature have occurred.

REC'D THIS IS 11/3/80
RECEIVED - 11/3/80

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

June 13, 1984
BECo Ltr. #84-080

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406


Docket No. 50-293
License DPR-35

Dear Sir:

The attached update Licensee Event Report 80-044/03X-1, "RCIC Isolation Valve," is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specification 6.9.B.2.b.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,


William D. Harrington

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Enclosure: LER 80-044/03X-1

cc: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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