UPDATE REPORT

PREVIOUS REPORT ISSUED 2/23/79

NR FORM 266 (12-51) 10 CFR 50	U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT	APPROVED BY OMB 3150-0011
CONTROL BLOCK:		
0 1 M A P P S 1 0		
CONT 0 1 REPORT L 6 0 5 0 -0 29 3 0 0 21 4 7 9 8 0 6 1 3 8 4 9 r source 50 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80		
Image: Construction of the second constructined consecond construction of the second construction		
Operability and Flow Rate Test @ 1000 psig), the HPCI turbine tripped. Im-		
[0 4] [mediate investigation revealed that the lower head gasket on the gland seal		
[0 5] [condenser had ruptured. The HPCI was declared INOP, and operability of backup		
of systems was immediately demonstrated. The event created no threat to the		
0 7 public health and safety.		
08		
7 8 SYETEM CAUSE CODE CODE	CAUSE COMPONENT CODE COM	P. VALVE
	$\textcircled{12} \boxed{[2]} \textcircled{13} \boxed{[X]} \times $	
Image: State of the state o		
ACTION FUTURE EFFECT SHL TAKEN ACTION ON PLANT MI A 18 Z 19 Z 20 Z 33 34 34 35 35 33 3	Line <thline< th=""> Line Line <thl< td=""><td>PRIME COMP. SUPPLIER A3 COMPONENT MANUFACTURER (0) (2) (2) (2) (2) (2) (4) (4) (4) (4) (4) (4) (4) (4</td></thl<></thline<>	PRIME COMP. SUPPLIER A3 COMPONENT MANUFACTURER (0) (2) (2) (2) (2) (2) (4) (4) (4) (4) (4) (4) (4) (4
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (2) Probable cause was attributed to excessive moisture entering an electrical box,		
1 resulting from a ruptured gland seal condenser gasket, causing the inadvertant		
1 2 [trip of the aux. oil pump. Repairs were completed, test 8.5.4.1 successfully		
13 performed, and HPCI was returned to service. A similar occurrence was identified		
14 Lin LER 82-24.		
7 8 9 PACILITY STATUS SPOWER OTH 1 5 E 28 1 0 0 29	N/A B 31 Operat	or Observation
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF A	ACTIVITY 33 LOCATION	OF RELEASE 36
7 8 9 10 11 PERSONNEL EXPOSURES NUMBER 1955 DESCRIPTIO	44 45 N(39)	
	F./	A
PERSONNEL INJURIES NUMBER DESCRIPTION (4)	N/A 8407020094 8404	13
7 8 9 11 12 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)	S	PDR **
	N/A	60
ISSUED DESCRIPTION (45)	N/A	NRC USE ONLY
7 8 8 10		69 69 80 (617) 746-7900
NAME OF PREPARER	-J. MARTITON PHONE	

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ATTACHMENT TO LER 79-008/03X-1

On February 14, 1979, while performing Surveillance Test 8.5.4.1 (HPCI Pump Operability Flow Rate and Valve Test @ 1000 psig), the HPCI turbine tripped. Immediate investigation revealed that the lower head gasket on the gland seal condenser had ruptured. The HPCI was declared inoperable, and operability of the backup system was immediately demonstrated.

The most probable cause of the turbine trip is attributed to the inadvertent trip of the auxiliary oil pump prior to the turbine reaching speed. It was determined at the time, that the gland seal condenser head gasket had ruptured due to excessive pressure, resulting from the out-of-calibration condition of Pressure Control Valve PCV 2301-46. Excessive moisture, caused by the ruptured gasket, entered an electrical control circuit box causing the inadvertant trip of the auxiliary oil pump.

Immediate corrective action involved installation of new gland seal condenser head gaskets, and recalibration of the pressure control valve. Surveillance Procedure 8.5.4.1 was satisfactorily performed, and the HPCI was declared operable on February 14, 1979.

To reduce the potential for future gasket failures due to pressure transients, Restricting Orifice RO-2301-60 was relocated to the inlet side of the gland seal condenser from the outlet side (refer to GE SIL No. 129). Also, to reduce the frequency of gasket failures, the upper and lower neoprene gaskets were removed and replaced with gaskets made of Chesterton 290 material.

A similar occurrence was identified in LER 82-24.

BOSTON EDISON COMPANY BOD BOYLSTON STREET BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON

June 13, 1984 BECo Ltr. #84-081

Dr. Thomas E. Murley Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avanue King of Prussia, PA 19406

> Docket Number 50-293 License DPR-35

Dear Sir:

The attached update Licensee Event Report 79-008/03X-1, HPCI Trip During Testing, is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specification 6.9.B.2.b.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W. D. Harrington

PH:ko

Enclosure: LER 79-008/03X-1

cc: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

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