SAFETY EVALUATION BY THE DIVISION OF REACTOR LICENSING DO NOT REMOVE

LOCKHEED AIRCRAFT CORPORATION - RADIATION EFFECTS REACTOR

PROPOSED CHANGE NO. 6

INTRODUCTION

By letter dated October 17, 1967, the Lockheed Aircraft Corporation proposed changes in the Technical Specifications of License No. R-86. The proposed changes are for the purposes of reducing the required pressure differential between the Operations Building and the atmosphere, and of correcting and clarifying other aspects of the present Technical Specifications. This change has been designated Proposed Change No. 6.

DISCUSSION

The Technical Specifications presently require a positive pressure differential of at least one inch of water between the Operations Building and the atmosphere. Maintaining a pressure differential of one inch of water would substantially restrict the air flow through the Operations Building and service tunnel to the Reactor Building, and would place undue loads on access doors. The licensee proposes to maintain a positive pressure differential of approximately 0.1 inch of water which ensures positive control of the environment in the Operations Building. An alarm will sound in the control room whenever the pressure differential between the Operations Building and the atmosphere decreases below the set point of 0.1 inch of water and appropriate corrective action will be taken when the alarm is activated. We consider this pressure differential to be acceptable.

The Technical Specifications further require that the "car ejector pump" be maintained on the emergency power system. The is ejection system, of which the car ejector pump was a part, provided a mechanism for moving flat cars carrying experiments away from the Reactor Building to permit a locomotive to approach them. The system has been removed. Therefore, emergency power for the car ejector pump is not necessary. We are in agreement with the licensee's proposal to delete this requirement from the Technical Specifications.

The licensee proposes further revisions to clarify that reactor coolant samples are required only while the reactor is shutdown. Sampling of

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the primary and secondary system coolant cannot be accomplished during reactor operation; however, monitoring of the 1-135 content in the primary system coolant is performed and a significant increase in the I-135 level is cause for a reactor shutdown. We are in agreement with the proposed changes as clarification of the original intent of the specifications.

The linensee has also proposed changes in the Technical Specifications which would (1) change the frequency of the Reactor Safety Committee meetings from quarterly to 4 times annually with elapsed time between meetings not to exceed 4 months, and (2) include the entire Lockheed. Georgia Nuclear Laboratories site in the practice emergency drills on an annual basis while maintaining the quarterly drill frequency for procedures involving only the reactor facility. The change in the frequency of the Committee meetings is requested in order that the Committee may schedule its meetings to accommodate review requirements, while assuring specific frequency of meetings. These changes are primarily for convenience and clarification. We believe that the effectiveness of the Reactor Safety Committee is not decreased and that the proposed frequencies for drills are reasonable; therefore, we consider these changes to be acceptable.

CONCLUSION

Based on the foregoing, we have concluded that the proposed changes can be made with reasonable assurance that the health and safety of the public will not be endangered.

Therefore, the Technical Specifications of License No. R-86 may be revised as shown on Attachment A.

Donald J. Ekovholt

Assistant Director for Reactor Operations

Division of Reactor Licensing

londo J Storbell

Date: May 16, 1968