

### UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20556-0001

## PUBLIC SERVICE ELECTRIC & GAS COMPANY ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

#### HOPE CREEK GENERATING STATION

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81 License No. NPF-57

- The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated January 11, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

 The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director Project Directorate I-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 25, 1995

# FACILITY OPERATING LICENSE NO. NPF-57 DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
ix	ix
xviii	xviii
3/4 3-103	3/4 3-103
3/4 3/104	3/4 3-104
B 3/4 3-7	B 3/4 3-7

#### INDEX

#### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

SECTION			PAGE
	Table 3.3.7.4-1	Remote Shutdown Monitoring	
		Instrumentation	3/4 3-75
	Table 3.3.7.4-2	Remote Shutdown Systems Controls	3/4 3-77
	Table 4.3.7.4-1	Remote Shutdown Monitoring Instrumentation Surveillance	
		Requirements	3/4 3-82
	Accident Monitor	ring Instrumentation	3/4 3-84
	Table 3.3.7.5-1	Accident Monitoring Instrumentation	3/4 3-85
	Table 4 3 7 5-1	Accident Monitoring Instrumentation	
	11010 1.3.7.3	Surveillance Requirements	3/4 3-87
	Source Range Mon	itors	3/4 3-88
	Traversing In-Co	ore Probe System	3/4 3-89
	Radioactive Liqu	id Effluent Monitoring Instrumentation	3/4 3-91
	Table 3 3 7 9-1	Radioactive Liquid Effluent	
	10016 3.3.7.3-1	Monitoring Instrumentation	3/4 3 00
		Monitoring instrumentation	3/4 3-32
	Table 4.3.7.9-1	Radioactive Liquid Effluent Monitoring	
		Instrumentation Surveillance	
		Requirements	3/4 3-94
	Radioactive Gase	ous Effluent Monitoring Instrumentation .	3/4 3-96
	Table 3 3 7 10-1	Radioactive Gaseous Effluent	
	10020 3.3.7.20-2	Monitoring Instrumentation	3/4 3-97
			2/2 2 2/
	Table 4.3.7.10-1	Radioactive Gaseous Effluent Monitoring	
		Instrumentation Surveillance	
		Requirements	3/4 3-10
3/4.3.8	DEESTED		3/4 3-10
3/4.3.9	FEEDWATER/MAIN T	URBINE TRIP SYSTEM ACTUATION	
		The state of the s	3/4 3-10
			3/4 3-40
	Table 3.3.9-1 F	eedwater/Main Turbine Trip System	
	A	ctuation Instrumentation	3/4 3-10

#### INDEX

B				

SECTION										P)	AGE	
INSTRUM	ENTATION (Continued)											
	Seismic Monitoring Instrumentation									В	3/4	3-4
	Meteorological Monitoring Instrumentation		*							B	3/4	3-4
	Remote Shutdown Monitoring Instrumentation											
	and Controls											
	The state of the s											
	Source Range Monitors											
	Traversing Tn-Core Probe System	*	*	*	ж.	*	*	*	*	B	3/4	3-5
	Radioactive Liquid Effluent Monitoring											
	Instrumentation	*	*	*	*	*	*	*	*	В	3/4	3-6
	Radicactive Gaseous Effluent Monitoring											
	Instrumentation			*	٧.	$\times$			4	B	3,4	3-6
3/4.3.8	DELETED		á	*	4	×	*			B	3/4	3-7
3/4.3.9	FEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATIO	NC										
	INSTRUMENTATION		×				*			B	3/4	3-7
	Figure B3/4 3-1 Reactor Vessel Water Level				ď,					В	3/4	3-8
3/4.4 F	EACTOR COOLANT SYSTEM											
3/4.4.1	RECIRCULATION SYSTEM						h			В	3/4	4-1
3/4.4.2	SAFETY/RELIEF VALVES	٠	*		٠	*	٩.	*	Ť.	В	3/4	4-la
3/4.4.3	REACTOR COOLANT SYSTEM LEAKAGE											
	Leakage Detection Systems		į.						ď	В	3/4	4-3
	Operational Leakage											
	operation and the second								-21		-/-	
3/4.4.4	CHEMISTRY	j.	×	×	÷		v.	ì	ă.	В	3/4	4 - 3
3/4.4.5	SPECIFIC ACTIVITY	v			,		4			В	3/4	4-4
3/4.4.6	PRESSURE/TEMPERATURE LIMITS									В	3/4	4-5
	Table B3/4.4.6-1 Reactor Vessel Toughness	14	*	*	Œ	×	*	*		В	3/4	4-7
	Figure B3/4.4.6-1 Fast Neutron Fluence											
	(E>1Mev) at (1/4)T as a											
	Function of Service life									B	3/4	4-8

#### INSTRUMENTATION

3/4.3.8 DELETED

THIS PAGE INTENTIONALLY BLANK

3/4.3.8 DELETED

#### 3/4.3.9 FEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATION INSTRUMENTATION

The feedwater/main turbine trip system actuation instrumentation is provided to initiate action of the feedwater system/main turbine trip system in the event of a high reactor vessel water level (Level 8) to mitigate potential damage to the main turbine.