

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
R.E. Ginna Nuclear Power Plant, Unit No. 1

DOCKET NUMBER (2)
0 5 0 0 0 2 4 4

PAGE (3)
1 OF 0 2

TITLE (4)
Automatic Actuation of the Engineered Safety Feature (ESF)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	5	2	2	8	4	8	4	0	0	6	0	0	0	6	2	2	8	4	0	5	0	0	0

OPERATING MODE (9) N

POWER LEVEL (10) 0 1 0 1 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

<input checked="" type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.408(e)	<input checked="" type="checkbox"/> 80.73(e)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.408(a)(1)(i)	<input type="checkbox"/> 80.38(e)(1)	<input type="checkbox"/> 80.73(e)(2)(v)	<input type="checkbox"/> 73.71(e)
<input type="checkbox"/> 20.408(a)(1)(ii)	<input type="checkbox"/> 80.38(e)(2)	<input type="checkbox"/> 80.73(e)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
<input type="checkbox"/> 20.408(a)(1)(iii)	<input type="checkbox"/> 80.73(e)(2)(i)	<input type="checkbox"/> 80.73(e)(2)(vii)(A)	
<input type="checkbox"/> 20.408(a)(1)(iv)	<input type="checkbox"/> 80.73(e)(2)(ii)	<input type="checkbox"/> 80.73(e)(2)(vii)(B)	
<input type="checkbox"/> 20.408(a)(1)(v)	<input type="checkbox"/> 80.73(e)(2)(iii)	<input type="checkbox"/> 80.73(e)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: G. F. Larizza, Operations Manager

TELEPHONE NUMBER: 3 1 5 5 2 4 - 4 4 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
A	A	B - - P T	X 9 9 9	N					
A	S	B - - P T	X 9 9 9	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 22, 1984, during Reactor Coolant System (RCS) heatup to hot shutdown from cold shutdown, the Automatic Safety Injection (S.I.) was actuated due to low steam line pressure while the primary system pressure was > 2000 psig with auto unblock of S.I. present. When the reason for the automatic actuation was diagnosed and verification was made that all the S.I. equipment and systems worked properly, S.I. was reset, terminated, and then realigned to normal conditions. RCS pressure was then reduced to 1750 psig. The auto S.I. was caused by operator error, compounded by some procedural inadequacy.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant Unit No. 1	DOCKET NUMBER (2) 0 5 0 0 0 2 4 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 0 6	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On May 22, 1984, during Reactor Coolant System (RCS) heatup to hot shutdown from cold shutdown, the Automatic Safety Injection (S.I.) was actuated due to low steam line pressure while the RCS pressure was > 2000 psig with auto unblock of S.I. present.

The immediate diagnosis by Operations personnel was that the Auto S.I. was generated by low steam generator steam line pressure (514 psig) coincident with > 2000 psig in RCS (auto S.I. unblock). The auto S.I. was caused by operator error, compounded by some procedural inadequacy. Emergency procedure E-1.1 "Immediate Action and Diagnostics for Spurious Actuation of S.I., LOCA, Loss of Secondary Coolant, and Steam Generator Tube Rupture" was consulted, although it did not apply for the plant condition that existed. Two licensed Reactor Operators concurred, as required by procedure, to terminate S.I. Verification was made that all the S.I. equipment functioned properly, S.I. was reset, terminated and realigned, and RCS pressure was reduced to 1750 psig.

The NRC was notified via telephone of the occurrence within four hours in accordance with 10 CFR 50.72.B.2.II.

To preclude reoccurrence, operating procedure O-1.1 "Plant Heatup from Cold Shutdown to Hot Shutdown" was modified. An existing precaution in the procedure has been repeated earlier in the procedure before exceeding RCS pressure of 1500 psig, to warn operators of potential S.I. actuation should RCS pressure go above 2000 psig while S/G steam line is < 514 psig. Operations personnel involved recognized their error in letting RCS pressure increase too fast when compared to secondary side steam pressure.



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001



ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

June 22, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: LER 84-006, Automatic Actuation of the Engineered Safety
Feature (ESF)

R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(iv), which requests a report of "any event or conditions that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS)," the attached Licensee Event Report LER 84-006 is hereby submitted.

Very truly yours,

Roger W. Kober

xc: U. S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

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