

LICENSEE EVENT REPORT

EXHIBIT A

NRC Form 366
(9-83)

U.S. Nuclear Regulatory Commission
Approved OMB No. 3150-0104
Expires: 8/31/85

L I C E N S E E E V E N T R E P O R T (L E R)

FACILITY NAME (1) Arkansas Nuclear One - Unit 2										DOCKET NUMBER (2) (PAGE (3)) 1015101010131618110F1012																																	
TITLE (4) Primary Overcurrent Protection Device on Containment Penetration Inoperable - LCO Exceeded																																											
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names		Docket Number(s)																																
01	91	01	51	71	81	41	--	01	11	21	--	01	01	61	11	81	81	41	10151010101																								
OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																											
POWER LEVEL (10)		20.402(b)		20.405(a)(1)(i)		20.405(a)(1)(ii)		20.405(a)(1)(iii)		20.405(a)(1)(iv)		20.405(a)(1)(v)		20.405(c)		50.36(c)(1)		50.36(c)(2)		50.73(a)(2)(i)		50.73(a)(2)(ii)		50.73(a)(2)(iii)		50.73(a)(2)(iv)		50.73(a)(2)(v)		50.73(a)(2)(vii)		50.73(a)(2)(viii)(A)		50.73(a)(2)(viii)(B)		50.73(a)(2)(x)		73.71(b)		73.71(c)		Other (Specify in Abstract below and in Text, NRC Form 366A)	
Name														LICENSEE CONTACT FOR THIS LER (12)				Telephone Number																									
Dan Moeggenberg, Licensing Engineer														Area		Code		15101191614131213161																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																											
Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS																																		
SUPPLEMENT REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		Month		Day		Year																							
[] Yes (If yes, complete Expected Submission Date) [X] No																																											
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																											

During a review of documentation to verify the correctness of a proposed technical specification change request regarding containment penetration overcurrent protection devices, an apparent error was noted in a vendor electrical drawing for reactor coolant system sampling cabinet 2C116. The drawing indicated that fuse protection, being relied upon as the primary penetration overcurrent protection device, was bypassed by jumpers. Since no other documentation was found that indicated the jumpers had been removed subsequent to receipt and installation of the cabinet, a special investigation was performed on 5/18/84, at 1620 hours to ascertain the existence of the jumpers. The jumpers were found to exist as indicated on the drawings. An engineering evaluation was promptly performed to allow removal of the jumpers, and the necessary wiring changes were completed by 1800 hours on 5/18/84.

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NRC Form 366A
(9-83)

Form 1062.01B
U.S. Nuclear Regulatory Commission
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (e)			PAGE (3)
		Year	Sequential	Revision	
			Number	Number	
Arkansas Nuclear One - Unit 2	0151010101 31 61 81 81 41--	0	1	2	-- 0 1 0 01210F1012

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 5/18/84, a special inspection was performed to resolve an apparent drawing error on a vendor print. The print indicated hard wired jumpers around power and control fusing to three DC powered solenoid valves in the reactor coolant (RCS) sampling system. The jumpers were contained in 2C116, RCS sampling cabinet, supplied by Delphi Industries. After visually verifying that the jumpers did in fact exist as shown by the print, an engineering evaluation was promptly conducted and rewiring instructions were provided to the maintenance staff. The jumpers were removed, restoring fuse protection, within approximately two hours. This is considered an isolated event, therefore, no additional action to reduce the probability of future occurrences is planned.

The fusing is relied upon as the primary device to provide overcurrent protection for containment penetration 2WR26-3. With the fusing in the circuit bypassed by the jumpers, 2D21 DC motor control center breaker 26 was still available to interrupt possible fault current to prevent damage to containment penetration 2WR26-3.

Since there is no record of a plant change to install the jumpers, it is believed that the jumpers were installed either during the fabrication or installation of the cabinet. As such, the error is considered a design/construction error that has existed since 1978. The event date, September 5, 1978, is the date when the unit first entered Mode 4 and Technical Specification 3.8.2.5 first became applicable. Because the Limiting Condition for Operation associated with Technical Specification 3.8.2.5 was exceeded, this event is reportable per 50.73(a)(2)(i)(b).

Another event regarding containment overcurrent protective devices (breakers) was reported in (50-368) LER 83-049/01T-0.



ARKANSAS POWER & LIGHT COMPANY

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June 18, 1984

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 84-012-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(i), attached is the subject report concerning an apparent error noted in a vendor electrical drawing for reactor coolant system sampling cabinet 2C116.

Very truly yours,

John R. Marshall
Manager, Licensing

JRM:JRS:ac

Attachment

cc: Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
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