

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of  
ARKANSAS POWER AND LIGHT COMPANY  
(Arkansas Nuclear One, Unit 1)

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Docket No. 50-313

ORDER CONFIRMING LICENSEE COMMITMENTS  
ON EMERGENCY RESPONSE CAPABILITY

I.

Arkansas Power and Light Company (AP&L or the licensee) is the holder of Facility Operating License No. DPR-51 which authorizes the operation of the Arkansas Nuclear One, Unit 1 (the facility) at steady-state power levels not in excess of 2568 megawatts thermal. The facility is a pressurized water reactor (PWR) located at the licensee's site in Pope County, Arkansas.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities and significant upgrading of emergency response capability based on the experience from the accident at TMI-2 and the official studies and investigations of the

accident. The requirements are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and in Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability." Among these requirements are a number of items consisting of emergency response facility operability, emergency procedure implementation, addition of instrumentation, possible control room design modifications, and specific information to be submitted.

On December 17, 1982, a letter (Generic Letter 82-33) was sent to all licensees of operating reactors, applicants for operating licenses, and holders of construction permits enclosing Supplement 1 to NUREG-0737. In this letter, operating reactor licensees and holders of construction permits were requested to furnish the following information, pursuant to 10 CFR 50.54(f), no later than April 15, 1983:

- (1) A proposed schedule for completing each of the basic requirements for the items identified in Supplement 1 to NUREG-0737, and
- (2) A description of plans for phased implementation and integration of emergency response activities including training.

### III.

AP&L responded to Generic Letter 82-33 by letter dated April 15, 1983. In this submittal, AP&L made commitments to complete the basic requirements. The attached Table summarizing AP&L's schedular commitments or status was developed by the NRC staff from the Generic Letter and the information provided by AP&L.

AP&L's commitments include (1) dates for providing required submittals to the NRC, (2) dates for providing schedules for providing required submittals to the NRC, (3) dates for implementing certain requirements, and (4) a schedule for providing implementation dates for other requirements. These latter implementation dates will be reviewed, negotiated and confirmed by a subsequent order.

The NRC staff reviewed AP&L's April 15, 1983 letter and entered into discussions with the licensee regarding schedules for meeting the requirements of Supplement 1 to NUREG-0737. As a result of these discussions, the NRC staff finds that the dates are reasonable, achievable dates for meeting the NRC requirements. The NRC staff concludes that the schedule proposed by the licensee will provide timely upgrading of the licensee's emergency response capability.

In view of the foregoing, I have determined that the implementation of AP&L's commitments is required in the interest of the public health and safety and should, therefore, be confirmed by an immediately effective Order.

#### IV.

Accordingly, pursuant to Sections 103, 1611, 1610 and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED, EFFECTIVE IMMEDIATELY, THAT the licensee shall:

Implement the specific items described in the Attachment to this ORDER in the manner described in AP&L's submittal noted in Section III herein no later than the dates in the Attachment.

Extensions of time for completing these items may be granted by the Director, Division of Licensing, for good cause shown.

V.

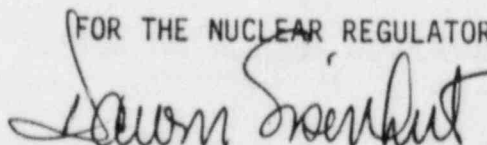
The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. Any request for a hearing should be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy should also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,  
this 14th day of June, 1984.

Attachment:  
Licensee's Commitments on  
Requirements Specified in  
Supplement 1 to NUREG-0737

LICENSEE'S COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

TITLE	REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)
1. Safety Parameter Display System (SPDS)	1a. Submit a safety analysis and an implementation plan to the NRC.	June 29, 1984 (1)
	1b. SPDS fully operational and operators trained.	Complete (2)
2. Detailed Control Room Design Review (DCRDR)	2a. Submit program plan to the NRC.	Complete
	2b. Submit a summary report to the NRC including a proposed schedule for implementation.	August 14, 1985
3. Regulatory Guide 1.97 - Application to Emergency Response Facilities	3a. Submit a report to the NRC describing how the requirements of Supplement 1 to NUREG-0737 have been or will be met.	June 29, 1984
	3b. Implement (installation or upgrade) requirements.	The implementation schedule will be provided in the report which will be submitted by June 29, 1984

(1) Will incorporate results of other Supplement 1 initiatives to determine needs for SPDS upgrade.

(2) Operational with existing parameters.

LICENSEE'S COMMITMENTS ON SUPPLEMENT I TO NUREG-0737

TITLE	REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)
4. Upgrade Emergency Operating Procedures (EOPs)	4a. Submit a Procedures Generation Package to the NRC.	Complete
	4b. Implement the upgraded EOPs.	Complete
5. Emergency Response Facilities	5a. Technical Support Center fully functional.	June 30, 1984 (1)
	5b. Operational Support Center fully functional.	Complete
	5c. Emergency Operations Facility fully functional.	June 30, 1984 (2)

(1) SPDS within the TSC will be operational with existing parameters.  
 (2) The licensee has requested exemption from locating the backup EOF 10 to 20 miles from ANO in favor of using the Russellville Local Office which is 7 miles from ANO. The acceptability of this request will be the subject of a separate licensing action.



UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of )  
 )  
 ARKANSAS POWER & LIGHT ) Docket No. 50-368  
 COMPANY )  
 )  
 (Arkansas Nuclear One, )  
 Unit 2) )

ORDER CONFIRMING LICENSEE COMMITMENTS  
ON EMERGENCY RESPONSE CAPABILITY

I.

Arkansas Power & Light Company (the licensee) is the holder of Facility Operating License No. NPF-6 which authorizes the operation of Arkansas Nuclear One, Unit 2 (the facility) at steady-state power levels not in excess of 2815 megawatts thermal. The facility is a pressurized water reactor (PWR) located at the licensee's site in Pope County, Arkansas.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities and significant upgrading of emergency response capability based on the experience from the accident at TMI-2 and the official studies and investigations of the

accident. The requirements are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and in Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability." Among these requirements are a number of items consisting of emergency response facility operability, emergency procedure implementation, addition of instrumentation, possible control room design modifications, and specific information to be submitted.

On December 17, 1982, a letter (Generic Letter 82-33) was sent to all licensees of operating reactors, applicants for operating licenses, and holders of construction permits enclosing Supplement 1 to NUREG-0737. In this letter operating reactor licensees and holders of construction permits were requested to furnish the following information, pursuant to 10 CFR 50.54(f), no later than April 15, 1983:

- (1) A proposed schedule for completing each of the basic requirements for the items identified in Supplement 1 to NUREG-0737, and
- (2) A description of plans for phased implementation and integration of emergency response activities including training.

### III.

Arkansas Power & Light Company (AP&L) responded to Generic Letter 82-33 by letter dated April 15, 1983. In this submittal, AP&L made commitments to complete the basic requirements. The attached Table summarizing AP&L's schedular commitments or status was developed by the NRC staff from the Generic Letter and the information provided by AP&L.



AP&L's commitments include (1) dates for providing required submittals to the NRC, (2) dates for implementing certain requirements, and (3) a schedule for providing implementation dates for other requirements. These latter implementation dates will be reviewed, negotiated and confirmed by a subsequent order.

The NRC staff reviewed AP&L's April 15, 1983 letter and entered into discussions with the licensee regarding schedules for meeting the requirements of Supplement 1 to NUREG-0737. As a result of these discussions, the NRC staff finds that the proposed dates are reasonable, achievable dates for meeting the Commission requirements. The NRC staff concludes that the schedule proposed by the licensee will provide timely upgrading of the licensee's emergency response capability.

In view of the foregoing, I have determined that the implementation of AP&L's commitments is required in the interest of the public health and safety and should, therefore, be confirmed by an immediately effective Order.

#### IV.

Accordingly, pursuant to Sections 103, 1611, 1610 and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED, EFFECTIVE IMMEDIATELY, THAT the licensees shall:

Implement the specific items described in the Attachment to this ORDER in the manner described in AP&L's submittals noted in Section III herein no later than the dates in the Attachment.

Extensions of time for completing these items may be granted by the Director, Division of Licensing, for good cause shown.

V.

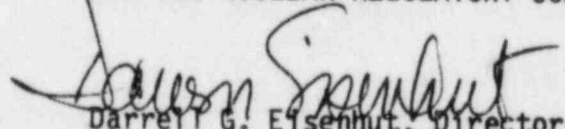
The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. Any request for a hearing should be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy should also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,  
this 14th day of June, 1984.

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Licensee's Commitments on  
Requirements Specified in  
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LICENSEE'S COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

TITLE	REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)
1. Safety Parameter Display System (SPDS)	1a. Submit a safety analysis and an implementation plan to the NRC.	April 15, 1984 C
	1b. SPDS fully operational and operators trained.	December 15, 1985
2. Detailed Control Room Design Review (DCRDR)	2a. Submit a program plan to the NRC.	Complete
	2b. Submit a summary report to the NRC including a proposed schedule for implementation.	May 5, 1986
3. Regulatory Guide 1.97 - Application to Emergency Response Facilities	3a. Submit a report to the NRC describing how the requirements of Supplement 1 to NUREG-0737 have been or will be met.	April 15, 1984 C
	3b. Implement (installation or upgrade) requirements.	To be provided with the April 15, 1984 report. C

LICENSEE'S COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

TITLE	REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)
4. Upgrade Emergency Operating Procedures (EOPs)	4a. Submit a Procedures Generation Package to the NRC.	Three months prior to the fourth refueling outage (estimated to commence on September 15, 1985).
	4b. Implement the upgraded EOPs.	During the fourth refueling outage (estimated to commence on September 1985).
5. Emergency Response Facilities	5a. Technical Support Center fully functional.	June 30, 1984 (1)
	5b. Operational Support Center fully functional.	Complete
	5c. Emergency Operations Facility fully functional.	June 30, 1984 (2)

(1) SPDS within the TSC will be operational with existing parameters.

(2) The licensee has requested exemption from locating the backup EOF 10 to 20 miles from AHO in favor of using the Russellville Local Office which is 7 miles from AHO. The acceptability of this request will be the subject of a separate licensing action.