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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OM8 NO 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)			LER NUMBER (6)	PAGE (3)	
Susquehanna Steam Electric Station			YEAR	SEQUENTIAL	REVISION		
Unit 1	0 5 0 0 3 8	17	8 4	- 0 2 6	- 010	0 2 0F	0 2

TEXT /// mane space is required, use additional NRC Form 386A's) (17)

AC Form 386A

Technical Specification Table 3.6.3-1 and FSAR Table 6.2-12 require Core Spray Full Flow Test Isolation Valves HV-152F015A, HV-152F015B (Unit 1) and HV-252F015A, HV-252F015B (Unit 2) to isolate on

1) Reactor Vessel Low Level 1, or

2) Primary Containment High Drywell Pressure

The As Built condition isolates the valves properly on .

1) Reactor Vessel Low Level 1.

The design and the As Built condition do not agree with the Technical Specifications or the FSAR in that these documents require an isolation on

2) A Primary Containment High Drywell Pressure.

The design and As Built condition both isolate the valves on Primary Contrinment High Drywell Pressure with a Low Reactor Pressure Permissive Signal.

Administrative control was placed on the full flow test valves by yellow tagging the valves closed. This will prevent the opening of these valves without the shift supervisor's knowledge.

The Core Spray Full Flow Test Valve Isolation Signal non-conformance was noted during a Technical Specification review, by utility personnel. Non-conformance Report (NCR) 84-747 and 84-748 were written to document the isolation signal descripency. Plant Modification Request (PMR) 84-3085 and 84-3086 are issued to modify the isolation logic. The modification involves removing the Low Reactor Pressure Permissive from the circuit, therefore, allowing the Core Spray Full Flow Test Valve to isolate on either High Drywell Pressure or Low Reactor Vessel Water Level only.

This occurrance was noted on May 16, 1984 at 1715. Reactor power level on Unit 1 was 100%. Unit 2 was <1%

The descrepancy in the isolation signals for the Core Spray Full Flow Test Isolation Signal will not effect the safe operation of the Nuclear Power Plant since the Core Spray System operability is not effected.



Pennsylvania Power & Light Company

June 14, 1984

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Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 84-026-00 ER 100450 FILE 841-23 PLA-2237

Docket No. 50-387 License No. NPF-14

Attached is Licensee Event Report 84-026. This event was determined reportable per 10CFR50.73(a)(2)(i) in that an isolation signal to the Core Spray Full Flow Test was not per Technical Specifications.

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H.W. Keiser Superintendent of Plant-Susquehanna

RWS/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
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King of Prussia, PA 19406

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