NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 2

TITLE (4)
Unscheduled Trip of the Control Room Emergency Filtration Units

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	2 6	8 4	8 4	— 0 5 0	— 0 0	0 6	2 8	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 1	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	50.72(b)(2)(ii)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
C.M. Powers, Reactor Engineering Supervisor	AREA CODE 5 0 9 — 3 7 7 - 2 5 0 1 Ext. 2996

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	I L	R A	K 0 2 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Control Room Emergency Filtration Unit (an ESF system) was automatically actuated due to a spike on the corresponding Control Room outside air radiation monitors.

After verifying that radiation levels were not above normal background, the radiation monitors and emergency filtration units were reset and returned to normal operation.

Subsequent investigation determined the cause of the spike to have been tripping of the reactor core isolation cooling turbine.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

a) Power Level - 3% (Thermal)
b) Plant Mode - 2

Event

The Control Room Emergency Filtration Unit (EPN: WMA-FN-54A) was automatically actuated on 5/26/84 by a High-High radiation alarm originating from the Control Room outside air intake radiation monitors (EPN: WOA-RIS-31A & 32A).

Immediate Corrective Action

Normal background radiation levels were observed at the monitors. The associated radiation recorder (EPN: WOA-RR-31) showed that monitors 31A and 32A had received a spurious spike of sufficient magnitude to trip the High-High radiation alarm. The alarms were promptly reset and the ESF system returned to normal.

Notification was given to the NRC in accordance with the requirements of 10CFR50.72(b)(2)(ii).

Further Corrective Action

It was determined that the spike originated from the reactor core isolation cooling turbine being tripped during testing. A Plant modification was initiated to install noise suppression devices in the associated control circuitry. Investigations and resolution of noise problems are continuing on the Process Radiation Monitoring and interfacing systems.

Safety Significance

There were no safety consequences associated with this event and all Plant systems performed as required during the event.

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397
June 22, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-050

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-050 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1900 hours on May 26, 1984.

Very truly yours,

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-050

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
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Walnut Creek, California 94596
Mr. A. D. Toth, NRC Resident Inspector (901A)
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