NRC F (9-83)		See											L	ıc	ENSE	E	EV	/EI	NT	RE	PORT	(L	ER)				U.S	AP	PROVE	REGUI D OME 8/31/8	NO.			ION
FACIL	TY	2 A MI	111		_	-		-		_		_	-	-			-	_	-	-		-		-	000	KET	NUM	BER	2)			,	AGE I	3)
					Nu	c16	ar	P	lar	nt.	-	Un'	it :	2															013	19	17	1 0	F O	12
TITLE	_		-		-	-	-	-	-	-	-		-			-			-	-		-		-		-					-			
Uns	sci	hed	u1	ed	I	nii	tia	ti	on	of	C	ont	tro	1 1	Room	Er	mei	rg	end	cy	Filtra	at	ion	Un	its									
-	_	WT DA	-	-				-	-	-	ER I	-	-		production in the last	-	RTD	-	-	T		-	-	-	-	-	ES IN	(VOL	VED (8	1)				
MONT	MONTH DAY YEAR				E	YE	AR	SEQUENTIAL MEN					REVE	HON	MONTH	T	DAY		YEAR		PACILITY NAMES					DOCKET NUMBER(S)								
	+	-			-				NUMBER			NUMBER	-	-	+		_	-										0 5	10	0	0	1	1	
0 !	5	2	8	8	4	8	4	_	0	5	2	-	0	0	0 6	2	1	2	8	4		-							0 5	10	0	0	1	L
-		ATIM	-	7		THE	SAEP	ORT	15.5	UBM	HTTE	D PU	RSUA	NT T	O THE	REG	UIRI	ME	NTE	OF 10	CFR 5 10	Che	ck one o	r more	of th	e foil	owing	j (11)						
	MODE (9) 3						20.402(b)						20.406(e) X 80.73						80.73(a)(2)(iv)						73.71(b)									
POI	WER	T	-				20.408(a)(1)(i) 50					50.35	50.35(e)(1) 50.73(e)(2)(v)						73.7					73,71(e)	71 (e)									
LEVEL 0,0 0			20 405(a)(1)(ii) 56					50.36	50.36(c)(2) 50.73(a)(2)(vii)						2) (vii)	X					THER	THER ISpecify in Abstract												
				-	-		20.4	406 ia	1177	(ii)					50.736	1(2	Hil						10.73(e)	21(wmi)	(A)				1	166A)				
					20 405(a)(1)(iv)					50.73(a)(2)(ii) 50.73(a)(2)(vii						21(viii)	(8)				50	.72	(b)	(2)	(11)								
							20.6	406 is	1(3)	w)					50,736	a)(2) (m)						73(a)	2)(x)										
														L	ICENSEE	co	NTA	CT	FOR	THIS	LER (12)	-												
NAME																										ARE	4 00	-	ELEPH	ONEN	UMB	EH		_
~ .										1																			2.	7 7		2	r. 0	. 1
C.1	٩.	Po	we	rs	,	Kea	ict	or	-	-		-	of owner, where the		uperv	_		_								-	0			7 7	-	4	510	
	_			COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS RE											REPO	юят (13) Ext. 2996																		
CAUS	USE SYSTEM		STEM		COMPONE		ENT		MANUFAC TURER		REPORTABLE TO NPROS		BLE			*		CA	AUSE	SYSTEM	COMPONENT		NENT		MANUFAC TURER		0	TO NPH						
В		I	L	1		R	Α	K	0	2	0		N										11	1		1	L	_						
		1								1									1				1 1	1		1	1	1						
-	-	_	-	-	-	-	-		S	Libbi	EME	NTAI	REP	ORT	EXPECT	ED	(14)	-	_						T	_		CTE		MO	нти	DAY	YE	AR
										and the same of th						72									7	5	UBM	18810 E 115	N					

A Control Room Emergency Filtration Unit (an ESF system) was automatically actuated due to a spike on the corresponding Control Room Outside Air Radiation Monitors.

After verifying that radiation levels were not above normal background, the radiation monitors and emergency filtration units were reset and returned to normal operation.

Subsequent investigation determined the cause of the spike to have been the closure of a reactor core isolation cooling valve (EPN: RCIC-V-13).

8406280142 840622 PDR ADOCK 05000397 S PDR

YES IT IN COMPLETE EXPECTED SUBMISSION DATE!

IE27/1

NRC Porm 388A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REQULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)			L	ER NUMBER (6	PAGE (3)				
			PASY		SEQUENTIAL	REVISION NUMBER				
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3	91	814	-	0151 2		90	01	2 OF	01:

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

- a) Plant Power 0%
- b) Plant Mode 3

Event

The Control Room Emergency Filtration Unit (WMA-FN-54A) was automatically actuated on 5-28-84 by a High-High radiation alarm originating from the outside air intake monitors (EPN: WOA-RIS-31A & 32A).

Immediate Corrective Action

Normal background radiation levels were observed at the monitors. The associated radiation recorder (WOA-RR-31) revealed that monitors 31A and 32A had received a spurious spike of sufficient magnitude to trip the High-High alarm. These alarms were promptly reset and the ESF system returned to normal.

Notification was given to the NRC in accordance with the requirements of 10CFR50.72(b)(2)(ii).

Further Corrective Action

It was determined that the spike originated from the actuation of a reactor core isolation cooling valve (RCIC-V-13). A Plant modification was initiated to install noise suppression devices in the control circuitry of this valve. Investigation and resolution of noise problems are continuing on the Radiation Monitoring and interfacing systems.

Safety Significance

There were no safety consequences associated with this event and all Plant systems performed as required during the event.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397 June 22, 1984

Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2

LICENSEE EVENT REPORT NO. 84-052

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-052 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1253 hours on May 28, 1984.

Very truly yours,

J.D. Martin (M/D 927M) WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 84-052

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
1450 Maria Lane
Walnut Creek, California 94596
Mr. A. D. Toth, NRC Resident Inspector (901A)
Ms. Dottie Sherman
American Nuclear Insurers
The Exchange Suite 245
270 Farmington Ave.
Farmington, CT 06032