

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 2 DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 1 OF 0 3 PAGE (3)

TITLE (4) Safety Relief Valve on SLC

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)				
0	5	2	5	8	4	8	4	0	0	5	0	0	0	0
0	5	2	5	8	4	8	4	0	0	5	0	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 0 0	20.406(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v) <input checked="" type="checkbox"/>	73.71(c)
	20.406(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME T. L. Elton, Acting Superintendent of Regulatory Compliance TELEPHONE NUMBER 9 1 2 3 6 7 + 1 7 8 5 1 AREA CODE

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD'S	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD'S
X	B R	R   V   L   2   6   5		Y					
X	B R	R   V   L   2   6   5		Y					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE)  NO  EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 5/25/84 during performance of the "STANDBY LIQUID CONTROL SYSTEM" procedure (HNP-2-6310), (with the reactor in cold shutdown) plant personnel determined that the Standby Liquid Control system's pressure relief valves lifted at a lower than expected pressure when they were bench tested. Consequently, in the event of a loss of coolant accident the standby liquid control system would not have injected its sodium pentaborate solution against a reactor vessel pressure in excess of 600 PSIG.

A design change was initiated which requires replacing the existing pressure relief valves with new pressure relief valves of a different model. One of the two pressure relief valves has been satisfactorily replaced. The second relief valve will be installed prior to loading fuel in the vessel.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR 8 4	SEQUENTIAL NUMBER - 0 0 5	REVISION NUMBER - 0 0 0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73 (a)(2)(v)(A) because it showed that the Standby Liquid Control system might not have been capable of shutting down the reactor, and maintaining it in safe shutdown.

On 05/25/84, during performance of the six month "STANDBY LIQUID CONTROL SYSTEM" procedure, HNP-2-6310, (with the reactor in cold shutdown), plant personnel determined that the Standby Liquid Control (SLC) system's pressure relief valves for loop "A" and loop "B" lifted at a pressure lower than the expected 1350 PSIG  $\pm$  25 PSIG when they were bench tested. Relief valve 2C41-F029A (loop A) lifted at 600 PSIG, and relief valve 2C41-F029B (loop B) lifted at 700 PSIG. These relief valves are positioned on the discharge side of the SLC pumps. When either valve lifts, the respective pump's discharge volume is then returned to the intake side of the SLC pump, thus completing a closed loop which recirculates the sodium pentaborate solution instead of injecting it into the reactor vessel.

In the event that a sufficient number of control rods could not have been inserted to control reactivity the SLC system would not have been able to inject into the reactor vessel against a pressure greater than 700 PSIG.

The control rod drive system was operable during this non-repetitive event. No justification for continued operation is required because the unit was in cold shutdown for recirculation piping repairs when this event occurred. This event had no impact upon any other system in Unit 2, or Unit 1.

After disassembly and examination it was determined that the valve stem in relief valve 2C41-F029A was broken at the cotter pin hole which allowed the valve spring to decompress and lose its tension. Relief valve 2C41-F029B was disassembled and found to have excessive wear on the valve stem and the valve stem guide.

A design change was initiated to replace both existing Lonergan model LCT 20 relief valves with Lonergan model LCT 30 relief valves. Relief valve 2C41-F029A (model LCT 30) was satisfactorily bench tested per HNP-2-6310 and installed on 06/08/84.

Relief valve 2C41-F029B (model LCT 30) has not been installed because it has not been received from the vendor. However, it will be tested per HNP-2-6310 and installed prior to loading fuel in the vessel.

When relief valve 2C41-F029B is received from the vendor, it will be bench tested per HNP-2-6310 and installed (promised date of vendor shipment is 06-20-84).

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

IDENTIFICATION OF EACH FAILED COMPONENT

MPL No.	Vendor	Model No.
2C41-F029A	LONERGAN	LCT-20
2C41-F029B	LONERGAN	LCT-20

Georgia Power Company  
Post Office Box 439  
Baxley, Georgia 31513  
Telephone 912 367-7781  
912 537-9444



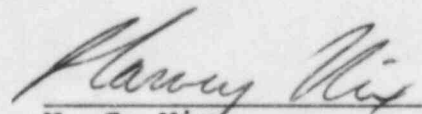
Edwin I. Hatch Nuclear Plant

June 23, 1984  
GM-84-537

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-05. This report is required by 10CFR 50.73(a)(2)(v)(A).

  
H. C. Nix  
General Manager

*see*  
HCN/TLE/vlt

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