UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Georgia Power Company, et al., (the licensee) dated March 10, 1982 and June 11, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 100 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Rivenbark, Acting Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 20, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
3.7-10a	3.7-10a
3.7-11	3.7-11
3.7-13	3.7-13
6-17	6-17

8. Shutdown Requirements

If Specification 3.7.A cannot be met, an orderly shutdown shall be initiated and the reactor shall be brought to Hot Shutdown within 12 hours and shall be in the Cold Shutdown condition within the following 24 hours.

B. Standby Gas Treatment System

1. Operability Requirements

A minimum of three (2 of 2 in Unit 1 and 1 of 2 in Unit 2) of the four independent standby gas treatment system trains shall be operable at all times when Unit 1 secondary containment integrity is required.

With one of the Unit 1 standby gas treatment systems inoperable, for any reason, Unit 1 reactor operation and fuel handling and/or handling of casks in the vicinity of the spent fuel pools is permissible for a period of seven (7) days provided that all active components in the remaining operable standby gas treatment systems in each unit (minimum of 1 in Unit 1 and 1 in Unit 2) shall be demonstrated to be operable within 4 hours, and daily thereafter.

B. Standby Gas Treatment System

1. Surveillance When System Operable

At least once per operating cycle, not to exceed 18 months, the following conditions shall be demonstrated:

a. Pressure drop across the combined HEPA filters and charcoal absorber banks is less than 6 inches of water at the system design flow rate (+10%, -0%).

b. Operability of inlet heater at rated power when tested in accordance with ANSI N510-1975.

c. Air distribution is uniform within 20% across the filter train when tested in accordance with N510-1975.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
B. Standby Gas Treatment System	B. Standby Gas Treatment System
1. Operability Requirements (Cont'd)	1. Surveillance When System Operable (Cont'd)
If the inoperable Unit 1 standby gas treatment system is not made fully operable within the seven (7) day period, the Unit 1 reactor shall be shutdown and placed in the cold shutdown condition within the next 36 hours and Unit 1 or Unit 2 fuel handling operations shall be terminated within 4 hours.	d. Automatic initiation of each train of the Unit 1 and Unit 2 standby gas treatment systems.
Unit 1 reactor operation and Unit 1 or Unit 2 fuel handling shall not be allowed if both of the Unit 1 standby gas treatment systems are inoperable or if both of the Unit 2 standby gas treatment systems are inoperable.	e. Manual operability of the by-pass valve for filter cooling.
3.7.B.2 Performance Requirements	2. Filter Testing
a. The results of the in-place DOP and halogentated hydrocarbon tests at design flows on HEPA filters and charcoal absorber banks shall show 99% DOP removal and 99% halogenated hydrocarbon removal when tested in accordance with ANSI N510-1975.	a. The tests and analysis shall be performed at least once per operating cycle, not to exceed 18 months, or after every 720 hours of system operation, or following painting, fire or chamical release in any ventilation zone communicating with the system.
b. The results of laboratory carbon sample analysis shall show 90% of radioactive methyl iodine removal when tested in accordance with RDT-M16-1T (80°C, 95% R.H.).	b. DOP testing shall be performed after each complete or partial replacement of the HEPA filter bank or after any structural maintenance on the system housing
c. Fans shall be shown to operate within +10% -0% design flow when tested in accordance with ANSI N510-1975.	c. Halogentated hydrocarbon testing shall be performed after each complete or partial replacement of the carcoal absorber bank or after any structural maintenance on the system housing.
	d. Each circuit shall be operated with the heaters on at least 10 hours every month.

4.7.C.1. Surveillance While Integrity Maintained (Cont'd)

b. Secondary containment capability to maintain a minimum 1/4-inch of water vacuum under calm wind (5 mph) conditions with each filter train flow rate not more than 4000 cfm shall be demonstrated at each refueling outage, prior to refueling.

3.7.C.2 Violation of Secondary Containment Integrity

a. Without Hatch-Unit 1 secondary containment integrity, restore Hatch - Unit 1 secondary containment integrity within 4 hours, or perform the following (as applicable):
   (1) Suspend irradiated fuel and/or fuel cask handling in the Hatch-Unit 1 secondary containment.
   (2) Be in at least Hot Shutdown within the next 12 hours and meet the Conditions of 3.7.C.1.a within the next 24 hours.

b. Without Hatch-Unit 1 secondary containment, refer to the following Hatch-Unit 2 Technical Specification, for LCO's to be followed for Hatch-Unit 2:
   (1) Section 3.6.5.1.
   (2) Section 3.9.5.1.

2. Surveillance After Integrity Violated

After a secondary containment violation is determined the standby gas treatment system will be operated immediately after the affected zones are isolated from the remainder of the secondary containment. The ability to maintain the remainder of the secondary containment at 1/4-inch of water vacuum pressure under calm (5 mph) wind conditions shall be confirmed.

D. Primary Containment Isolation Valves

1. Valves Required to be Operable

During reactor power operation, all primary containment isolation valves listed in Table 3.7-1 and all reactor coolant system instrument line excess flow check valves shall be operable except as stated in Specification 3.7.D.2.

D. Primary Containment Isolation Valves

1. Surveillance of Operable Valves

Surveillance of the primary containment isolation valves shall be performed as follows:

a. At least once per operating cycle the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and the closure times specified in Table 3.7-1.

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP (Continued)

g. Conditions arising from natural or man-made events that, as a direct result of the event require unit shutdown, operation of safety systems, or other protective measures required by Technical Specifications.

h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

i. Performance of structures, systems, or components which requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during unit life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

THIRTY DAY WRITTEN REPORTS

6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event*. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

*All Type B and Type C Leakage Tests (i.e., Local Leak Rate Tests) that fail (i.e., test leakage is such that an LER would be required) during an outage shall be reported per one thirty-day written report and shall be submitted within 30 days of the end of such an outage.

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GEORGIA POWER COMPANY
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MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated March 10, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Rivenbark

George W. Rivenbark, Acting Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 20, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following page 6.16 of the Appendix "A" Technical Specifications with the enclosed page 6-16. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP (Continued)

g. Conditions arising from natural or man-made events that, as a direct result of the event require unit shutdown, operation of safety systems, or other protective measures required by Technical Specifications.

h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

i. Performance of structures, systems, or components which requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during unit life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

THIRTY DAY WRITTEN REPORTS

6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event*. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

*All Type B and Type C Leakage Tests (i.e., Local Leak Rate Tests) that fail (i.e., test leakage is such that an LER would be required) during an outage shall be reported per one thirty-day written report and shall be submitted within 30 days of the end of such an outage.