



GPU Nuclear Corporation
One Upper Pond Road
Parsippany, New Jersey 07054
201-316-7000
TELEX 136-482
Writer's Direct Dial Number:

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U. S. Nuclear Regulatory Commission
Att: Document Control Desk
Washington, DC 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station (OCNGS)
Docket No. 50-219
Facility Operating License No. DPR-16
Drywell Corrosion Monitoring Program

- References:
- (1) GPU Nuclear Letter C321-92-2163, "Oyster Creek Drywell Containment," May 26, 1992.
 - (2) NRC Letter dated June 30, 1992, "Oyster Creek Drywell Containment."
 - (3) GPU Nuclear Letter C321-93-2100, "Oyster Creek Drywell Inspection," March 25, 1993.

In compliance with Item (3) of References 1 and 2, and Reference 3, GPU Nuclear has (1) assessed the condition of the drywell based upon inspections performed at Oyster Creek during the 15R Outage and is (2) submitting an extended drywell inspection plan for the remaining life of the plant. GPU Nuclear remains committed, as stated in Reference 1, to continue taking drywell thickness measurements for the life of the plant.

Through the 15R Outage, GPU Nuclear's drywell containment vessel thickness monitoring program, Item (2) of References 1 and 2, consisted of ultrasonic thickness (UT) measurements taken at the sandbed region and upper elevations (cylinder, sphere) of the drywell during refueling outages and other outages of opportunity.

Assessment of the most recent UT data taken during the 15R Outage has determined that there is no evidence of ongoing corrosion in the upper elevations of the drywell and that corrosion has been arrested in the sandbed region of the drywell which was cleaned of sand and rust and coated during the 14R Outage (December 1992). The attached table summarizes the 15R Outage UT inspection results for both the sandbed region and upper

ADD 1

surfaces indicates that, after 21 months of service, the coating is performing satisfactory with no signs of deterioration such as blisters, flakes, discoloration, etc.

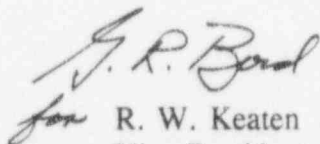
GPU Nuclear's extended inspection plan for the Oyster Creek drywell containment vessel covers both the upper elevations of the drywell and the coated sandbed region.

For the upper elevations of the drywell, this program will perform UT measurements during the 16R Outage (currently scheduled to begin September, 1996) and, as a minimum, again during every other refueling outage (18R, 20R, etc.). The UT measurement locations will be the nine areas identified as most severely corroded. Assessment of the most recent UT data taken during the 15R Outage has determined (and will be reconfirmed by the 16R inspections) that there is no evidence of ongoing corrosion in the upper elevations of the drywell. After each inspection, a technical assessment of the drywell condition will be made, any appropriate corrective action will be taken, and any necessary additional inspections would be scheduled to ensure that drywell integrity is maintained for the remaining life of the plant.

For the sandbed region of the drywell, this program will perform visual inspection of the external epoxy coating during the 16R Outage and, as a minimum, again during the 18R Outage (year 2000). The epoxy coating has an estimated life of 8-10 years which makes the current projected end of life between December, 2000 and December, 2002. Coating inspection shall be by direct (physical) and/or remote methods on a sample basis. Based upon these inspections, a technical assessment of the coating condition will be made, any appropriate corrective action will be taken, and the need for additional (post 18R) inspections will be determined to ensure that drywell integrity is maintained for the remaining life of the plant. In addition, while not technically required based upon the performance of the epoxy coating, UT thickness measurements will be taken one more time in the sandbed region during the 16R Outage, to the same extent as the 15R Outage inspections.

In compliance with Reference 3, GPU Nuclear remains committed to inform the NRC prior to implementing any changes to this drywell inspection program.

Very truly yours,



for R. W. Keaten
Vice President and Director
Technical Functions

Attachment
RTZ/plp

c: Administrator, Region 1
Senior Resident Inspector
Oyster Creek NRC Project Manager

TABLE 1

ACCEPTABLE MEAN DRYWELL THICKNESSES				
	15R OUTAGE INSPECTION DRYWELL THICKNESSES			
LOCATION	NOMINAL	UT MEASURED MINIMUMS (1)	CODE REQUIRED	MARGIN
Sandbed Region	1.154"	0.806"	.736" (2)	.070" (3)
Sphere (el. 50' - 2")	0.770"	0.733"	0.541"	0.192"
Sphere (el. 51' - 10")	0.722"	0.695"	0.518"	0.177"
Sphere (el. 60' - 11")	0.722"	0.709"	0.518"	0.191"
Cylinder (el. 87' - 5")	0.640"	0.613"	0.452"	0.161"

- (1) Thinnest Location as measured during the 15R outage, September, 1994.
- (2) Controlled by buckling.
- (3) Corrosion arrested (sandbed region coated in 14R outage).