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Approval
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Vogtle Electric Generating Plant
NUCLEAR OPERATIONS
Unit COMMON



Georgia Power

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FOR INFORMATION ONLY

WORK REQUEST PROGRAM

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1.0 PURPOSE

This procedure establishes administrative controls for maintenance activities at Vogtle Electric Generating Plant (VEGP). It provides for identification, control, and documentation of maintenance activities.

Procedure 29402-C, "WPG Work Request Processing" will be used in conjunction with this procedure to complete define the processing of a work order.

2.0 DEFINITIONS

2.1 WORK PLANNING GROUP (WPG)

A group of assigned individuals in the Planning, Scheduling and Work Control section of the Outages and Planning Department that plan, develop, prioritize, schedule, review, evaluate, and maintain history of repairs for Maintenance Work Orders (MWO).

2.2 PREVENTIVE MAINTENANCE (PM)

Work tasks performed on a predetermined schedule, in accordance with Procedure 20015-C, "Preventive Maintenance", to maintain equipment reliability.

2.3 PREDICTIVE MAINTENANCE (MP)

Work tasks performed on equipment or components to predetermine failures by obtaining and trending historical data through preventive maintenance, oil analysis, MOVATS, vibration monitoring corrective work orders, etc.

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- 2.4 PLANNED MAINTENANCE
- Preventive maintenance, predictive maintenance and other work tasks performed on equipment components to prevent unexpected failure.
- 2.5 CORRECTIVE MAINTENANCE
- Work tasks performed on systems or components to resolve items identified through preventive or predictive maintenance, surveillance, inspections and/or other methods.
- 2.6 WORK REQUEST TAG (WRT)
- A three part tag used to identify deficiencies or work required on plant equipment or components, to properly flag equipment in the field and to allow Shift Supervisor (SS)/On-Shift Operations Supervisor (OSOS) notification of equipment problems.
- 2.7 WORK ORDER (WO)
- When referenced in this procedure the step applies to an MWO and/or SWO.
- 2.8 MAINTENANCE WORK ORDER (MWO)
- A site document used to perform and document maintenance on plant equipment or components, to ensure specific work controls are identified and the work history is maintained.
- 2.9 SUPPORT WORK ORDER (SWO)
- A site document used to request support activities other than those activities encompassed in the maintenance work order program. Some examples of support activities are identified in Section 4.1.11.
- 2.10 SPECIAL INDICATORS
- Indicators identified in NPMIS equipment file to flag special requirements for applicable components. Some examples are critical components (RXTRIP), Technical Specification (TSPEC), Local Leak Rate Test (LLRT), etc.

2.11 WALKDOWN SHEET

A computer generated form, attached to each MWO or SWO initiated, to identify certain requirements for MWO or SWO packaging. A planning tool to identify drawings, manuals, special requirements, etc.

2.12 EMERGENCY MAINTENANCE

Any immediate mandatory maintenance or repair activity that is necessary to maintain safe operation or shutdown capabilities.

2.13 URGENT MAINTENANCE

Maintenance that is not as critical as emergency maintenance but in the opinion of the On-Shift Operations Supervisor is critical to schedule or safety and should be performed during the next 24-hour period.

2.14 CALIBRATION

Work tasks performed to compare and adjust equipment and/or instruments to a predetermined reference value, within tolerances.

2.15 SURVEILLANCE

Testing performed on a periodic basis to verify and document that structures, systems, and components are functioning properly and should remain in a readiness state capable of fulfilling the intended safety-related function.

2.16 FUNCTIONAL TESTS

Performance of those steps necessary to determine structures, systems and components function in accordance with predetermined specifications. Functional tests may include: Surveillance tests, ISI tests, ASME Section XI requirements and inspections in accordance with Procedure 29401-C, "Maintenance Work Order Functional Test".

2.17 INSPECTIONS

Examination, observation or measurement to determine the conformance of materials, supplies, components, parts, appurtenances, systems, personnel performance, procedures, processes or structures to predetermined requirements.

2.18 MAINTENANCE HISTORY

A written chronological record of work tasks performed on a component from initial turnover to present.

2.19 SUPERVISOR

A member of Georgia Power Supervision, foreman and above including a cognizant individual as defined in Procedure 00801-C, "Control Of Onsite Contractors".

2.20 CONTRACTOR MAINTENANCE

Maintenance performed on the plant by personnel selected according to Procedure 00801-C.

2.21 AUTHORIZED INSPECTOR

An employee of an Authorized Inspection Agency who has qualifications for, and has been properly qualified for ASME Section III Division 1 and ASME Section XI Preservice/Inservice Inspection.

2.22 SPECIAL REVIEW

A review by one of the following:

- Plant Review Board
- Fire Protection Engineer
- Equipment Qualification Group
- Technical Support
- Authorized Inspector (AI)

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2.21 FIRE PROTECTION PROGRAM COMPONENT

Those plant components required by VEGP FSAR 9.5.1 to be operational in order to maintain the viability of the Fire Protection Program.

2.22 CRITICAL COMPONENTS

Components which have been identified by reactor trip (RXTRIP) special indicator in NPMIS to trip the unit unless special care is taken. A system, control, protection scheme, or component which will cause a unit trip if it itself is inadvertently bumped, grounded, shorted, or misoperated, or if it fails in service.

3.0 RESPONSIBILITIES

3.1 MANAGER MAINTENANCE

The Manager Maintenance ensures that the Maintenance Program is effectively implemented as follows:

- 3.1.1 Policies, procedures, and administrative controls are established and updated as necessary.
- 3.1.2 Work tasks are documented and performed in accordance with approved procedures and safe work practices.
- 3.1.3 Installation of temporary jumpers and lifting wires in accordance with Procedures 00306-C, "Temporary Jumper And Lifted Wire Control" and 20429-C, "Short Term Documentation Of Temporary Jumpers And Lifted Wires".
- 3.1.4 Implementation of temporary modifications and hanging tags in accordance with Procedure 00307-C, "Temporary Modifications".
- 3.1.5 Disposition of parts/equipment removed from the plant is performed and documented.
- 3.1.6 Cleanliness requirements are established in accordance with Procedures 00254-C, "Plant Housekeeping/Material Condition Program" and 20427-C, "Maintenance Cleanliness And Housekeeping Control".

- 3.1.7 An On-the-Job Training (OJT) Program is developed and coordinated with the Training Department.
- 3.1.8 Mockups, when needed, are set up to hold work task exposures ALARA. Mockups are coordinated with Health Physics.
- 3.1.9 Ensure ASME Section XI Repair and Replacement Program is implemented in accordance with Procedure 20100-C, "ASME Section XI Repair/Replacement Program".
- 3.1.10 Ensure that a preventive maintenance program is established and implemented in accordance with Procedure 20015-C, "Preventive Maintenance".
- 3.1.11 Ensure that a predictive maintenance program is established and implemented in accordance with Procedure 20016-C, "Predictive Maintenance Program".
- 3.1.12 Nuclear Plant Management Information System (NPMIS) is operational to support all scheduled outage plans and the corrective, planned and surveillance maintenance programs.
- 3.2 MANAGER OPERATIONS
- The Manager Operations ensures that the Maintenance Program is supported as follows:
- 3.2.1 Operations and maintenance activities are coordinated effectively in accordance with existing plant conditions through the operations representative in the Work Planning Group (WPG).
- 3.2.2 WRTs are processed during off-normal working hours for emergency maintenance.
- 3.2.3 System status is recorded during work tasks when required.
- 3.2.4 Clearances are performed for work tasks.
- 3.2.5 Fire Protection (FP) Limiting Conditions for Operation (LCO) action statements are implemented prior to existence of the Limiting Condition to prevent violation of FP operability requirements.
- 3.2.6 Numbers are assigned for temporary jumpers and lifted wires in accordance with Procedure "00306-C, "Temporary Jumper And Lifted Wire Control".

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- 3.2.7 Functional test requirements may be assigned and/or changed by a qualified SS or OSOS.
- 3.2.8 All security-related work request are coordinated with the Security Department.
- 3.2.9 Assigns a VEGP licensed certified Operations Department Representative to authorize work that does not affect plant operations.
- 3.2.10 Ensures that Unit Shift Supervisor is cognizant of ongoing work on critical components.
- 3.2.11 Ensures Operations Engineering maintains the critical component List and an updated list is provided to Maintenance Engineering when any changes are made to the List for inclusion into NPMIS.
- 3.2.12 Transient combustible permits or ignition source permits are issued, as required and that the need for a fire watch has been evaluated.
- 3.3 **MANAGER ENGINEERING SUPPORT (MES)**
- The MES ensures that the Maintenance Program is supported as follows:
- 3.3.1 WRTs are initiated for Plant Modification Packages.
- 3.3.2 Designated work task procedures are provided for Plant Modification Packages.
- 3.3.3 Fire Protection post-work reviews, if applicable, are completed.
- 3.3.4 Shield plug and blockwall removal evaluation is completed if applicable.
- 3.3.5 Scaffolding pre-installation reviews, if applicable, are completed per approved plant procedures for scaffolding control.
- 3.3.6 Numbers are assigned for Temporary Modifications in accordance with Procedure 00307-C, "Temporary Modifications".

- 3.3.7 Engineering Support performs reviews required to support and improvement work. Specifically, the HVAC Supervisor/Duty Engineer completes reviews and grants approval of work activities such as welding and painting in areas that can affect exhaust and filtration charcoal beds.
- 3.4 WORK PLANNING AND CONTROLS SUPERINTENDENT
- The Work Planning and Controls Superintendent of the Outages and Planning Department ensures that the Maintenance Program is supported as follows:
- 3.4.1 Work orders receive all required reviews prior to issuing to maintenance.
- 3.4.2 RWP requests are submitted to Health Physics for work in Radiation Control Areas (RCA) usually 24 hours prior to starting work in accordance with Procedure 00930-C, "Radiation And Contamination Control".
- 3.4.3 MWO packages are assembled from WRT'S.
- 3.4.4 MWO packages are reviewed upon completion.
- 3.4.5 Equipment history records are maintained.
- 3.4.6 Maintenance activities are tracked.
- 3.4.7 Maintenance activities are accurately reported.
- 3.4.8 Maintenance work tasks are adequately defined and correctly resolved.
- 3.4.9 MWO packages are evaluated to ensure that acceptance criteria is met.
- 3.4.10 Maintenance work schedules are established.
- 3.4.11 Clearances are initiated for MWO packages.
- 3.4.12 Fire protection work evaluation is performed on all work orders by qualified personnel.
- 3.4.13 Identify MWO as critical component from the reactor trip possible special indicator in NPMIS.
- 3.4.14 Operations Work Planner assigns appropriate functional test requirements.

3.5 MANAGER PLANT TRAINING AND EMERGENCY PREPAREDNESS

The Manager Plant Training And Emergency Preparedness ensures that the Maintenance Program is supported as follows:

- 3.5.1 The OJT Program is coordinated with the Maintenance Department.
- 3.5.2 Adequate / training is provided to keep exposures ALARA as required.
- 3.5.3 Develop Training Programs to meet job qualification requirements established by the Maintenance Department and Outages and Planning Department.
- 3.5.4 Training records are maintained.

3.6 MANAGER HEALTH PHYSICS AND CHEMISTRY

The Manager Health Physics/Chemistry ensures that the Maintenance Program is supported as follows:

- 3.6.1 ALARA reviews are performed on all Radiation Work Permit (RWP) requests per Procedure 00930-C, "Radiation And Contamination Control", and Procedure 00910-C, "VEGP ALARA Program".
- 3.6.2 HP surveys are performed in a timely manner.
- 3.6.3 RWPs are issued per Procedure 43007-C, "Issuance, Use And Control Of Radiation Work Permits" and that proper radiological controls are instituted per 00910-C.
- 3.6.4 When contacted regarding draining of systems (Section 4.1.23) Chemistry is responsible for:
 - a. Evaluating the effect the draining will have on permitted effluent pathways.
 - b. Inspecting the system to evaluate the effectiveness of the corrosion control program, if time permits.
- 3.6.5 Provide for initial reviews of MWO's and SWO's.

3.7 MAINTENANCE ENGINEERING SUPERVISOR

The Maintenance Engineering Supervisor ensures that the Maintenance Program is supported as follows:

- 3.7.1 Predictive maintenance program work tasks are scheduled, tracked, trended, and evaluated.
- 3.7.2 Preventive maintenance program is maintained. Preventive or Predictive MWO's generated are evaluated to ensure train separation for equipment has been achieved prior to MWO generation.
- 3.7.3 A welding program is maintained in accordance with Procedure 20110-C, "Weld Control Program".
- 3.7.4 NPMIS equipment data base is maintained and periodic coordination with the WPG is conducted to determine need for additional fields for outage support activities.
- 3.7.5 Ensure ASME Section XI Repair and Replacement Program is implemented in accordance with Procedure 20100-C, "ASME Section XI Repair/Replacement Program".

3.8 QUALITY CONTROL SUPERINTENDENT

The Quality Control Superintendent ensures that the Maintenance Program is verified as follows:

- 3.8.1 Inspections, examinations and tests are performed in accordance with Procedure 00201-C, "Quality Control Inspection Program".
- 3.8.2 MWOs are reviewed for quality requirements.
- 3.8.3 Monitoring of work tasks is performed.

3.9 MANAGER OUTAGES AND PLANNING

The Manager Outages and Planning ensures planned outage, forced outage, and refueling activities effectively implement planning, scheduling, and maintenance program requirements as follows:

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- 3.9.1 Provides and approves for use all outage planning sequence of events prior to schedule distribution and implementation.
- 3.9.2 Work will be administratively governed by the Plant Administrative, Work Planning, and Outages and Planning procedures.
- 3.9.3 Work plans, procedures and practices shall be under the Nuclear Operations QA Program.
- 3.9.4 Work Orders assigned are packaged, planned, and scheduled by the Modifications and Outage Support Group (MOSG) and reviewed and approved by Work Planning Group.

3.10 MODIFICATIONS AND OUTAGE SUPPORT GROUP (MOSG)
SUPERINTENDENT

The MOSG Superintendent is responsible for certain maintenance support activities and implementing work package, on request.

4.0 INSTRUCTIONS

4.1 WORK REQUEST TAG INSTRUCTIONS

- 4.1.1 When conditions requiring maintenance are identified all plant personnel are responsible for completing a Work Request Tag (WRT). (See Figure 1 for Example)
- 4.1.2 A WRT may be submitted by any individual requesting maintenance support by completing the applicable portions of the WRT and delivering the WRT to the Support Shift Supervisor.

- 4.1.3 When completing the WRT the initiator should give an accurate, concise and complete description of the problem, indications for investigative maintenance, failure description for corrective maintenance, or a summary of the modification/installation for design changes in the problem block. This narrative should provide a clear description without the need to resort to reference documents, i.e., RER's, FCR's, etc.
- 4.1.4 If the WRT is to correct an item identified by a DC, NRC or QA Audit Finding or to address a commitment (regulatory or other) the initiator will indicate so by entering the identifying number in the problem block in addition to the problem description and attach a copy of the commitment, DC, etc. If the WRT was identified during the performance of a surveillance or an MWO, that document number will be referenced.
- 4.1.5 If the WRT is to implement a capital budget item, the general work order (GWO) number should be included in the problem block.
- 4.1.6 The initiator will complete the WRT through the "originator" line, attach the field copy to the component and deliver the WRT to the appropriate shift supervisor or WPG, as appropriate.

Non-corrective maintenance WRT's may be approved and processed by the WPG. (Examples are WRT's for DCP implementation, WRTs to perform planned activities to support outage preparation, etc). If processed by the WPG, the Operations Work Planning reviewer will complete the reviews and sign the OSOS/SS review block, in lieu of the actions required in section 4.1.7.

NOTES

- a. WRT's will be hung for items identified as needing corrective maintenance in the field. Items that are not physically broken, leaking, missing, etc... do NOT require the field copy WRT to be hung. Example of this type may be a WRT for a DCP.
- b. WRT's will NOT be hung on equipment inside the containment.

- 4.1.7 The Shift Supervisor will perform the following:
- 4.1.7.1 Evaluate the WRT for operational deficiencies which require reportability. (The On-Shift Operations Supervisor (OSOS) will make immediate notification to regulatory agencies and ensure a Deficiency Card is initiated in accordance with Procedure 00150-C.)
- 4.1.7.2 Evaluate the affect on plant operation and initiate compensatory action as required.
- 4.1.7.3 Evaluate the WRT for completeness. If any portion is determined inadequate or lacking in information, he will contact the initiator and/or return the WRT for clarification.
- 4.1.7.4 Assign the appropriate priority as defined below:
- X - Emergency (Sec. 2.11)
 - U - Urgent (Sec. 2.12)
 - 1 - Safety - Personnel, Plant Equipment, Public; LCO or equipment problem which could require a power reduction within 72 hours. Items with a significant impact on plant operation/efficiency.
 - 2 - Other LCO's with 7 day or less shutdown requirements. Items which impact continued plant operation/efficiency or that support surveillances.
 - 3 - Other LCO's, other items affecting plant operations/efficiency. Security, Fire Protection, License Commitments.
 - 4 - Items affecting plant material condition.
 - 5 - Items which may be repaired at any time.
- 4.1.7.5 Indicate associated LCO, Information LCO or Fire Protection LCO numbers in spaces provided.
- 4.1.7.6 Indicate the appropriate mode restraint.

NOTE

The mode restraint evaluation must be made conservatively. If a WRT is written for a problem which would not render a component inoperable, it should still be coded as a restraint to the lowest mode for which that component could be required operable. This ensures a proper evaluation is performed on each WRT immediately prior to a mode change. (In-progress maintenance activities could render a component temporarily inoperable or completed activities could require performance of a functional test prior to a mode change.)

- 4.1.7.7 Indicate if the work may be performed at power or during a unit outage. If outage, the highest mode in which the work may be performed should be indicated (i.e., 1 would indicate it is outage but may be performed in Mode 1 such as a MFWP outage WRT.)
- 4.1.7.8 Indicate special conditions required to perform maintenance by circling the appropriate 2 digit number on the back of the original tag.
- 4.1.7.9 Approve the WRT by signing and dating the WRT forwarding the original WRT to the Work Planning Group and a copy of the WRT to the Engineering Support Department for information.
- 4.1.8 The Work Planning Group (WPG) will process the WRT in accordance with 29402-C.

- 4.1.9 The WPG will determine if the WRT should be processed as a Maintenance Work Order (MWO) or a Support Work Order (SWO).

NOTE

Support Work Orders (SWO's) are generally work items that require less planning and/or administrative controls.

- 4.1.10 The following maintenance activities may be addressed without initiation of a Work Request Tag. If a WRT is initiated, it will be processed as a Support Work Order requiring minimal review prior to issuance.
- 4.1.10.1 Replacement of light bulbs, both indication and illumination bulbs.
- 4.1.10.2 Emergency and exit lighting, plant lighting fixtures and field run non EE580 related cabling.
- 4.1.10.3 Maintenance work on non-plant operational equipment by contractors under a maintenance agreement contract given out in accordance with Procedure 70106-C, "Contract Administration".
- 4.1.10.4 Labor items (pumping out ditches or manholes, spill cleanup, transporting barrels, shoring, cribbing, etc.)
- 4.1.10.5 Inking systems on plant chart recorders.
- 4.1.10.6 Security System Camera adjustments and cleaning as requested by the SNS.

CAUTION

Prior to performing any of the following (4.1.10.7 through 4.1.10.24), notify the Unit Shift Supervisor. Notification is also required at the start of each shift, unless the USS indicates otherwise.

- 4.1.10.7 Installing and/or changing identification tags except for ASME Code plates and vendor name plates.

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- 4.1.10.8 Replacement of passive accessories, such as knobs, handles, latches (for non-seismic equipment), lens covers (except for those required for Equipment Qualification), thumb screws, etc.
- 4.1.10.9 Tightening of packing on manually operated valves (Motor- and air-operated valve packing adjustments and repacking a valve require a WRT and MWO.)
- 4.1.10.10 Tightening of flanged leaks, fitting leaks, etc. on non-safety-related piping and/or tubing.
- 4.1.10.11 Instrument calibrations when required by approved plant procedures for instrument calibrations and/or adjustments when performed using approved plant procedures that require SS notification prior to and or completion of the work.

When performed during maintenance activities other than surveillances, all pertinent documentation shall be transmitted to Document Control and filed by procedure number.
- 4.1.10.12 Investigative-type work that assists in the determination of problem identification or description. Investigative work may include use of instruments (Fluke meters, VOM, etc.) but does not include component disassembly, lifted leads or other corrective-type maintenance.
- 4.1.10.13 Maintenance activities performed during the performance of approved plant surveillance procedures providing all the requirements of this procedure for the maintenance activity, such as QC review, Equipment Qualification, parts traceability, etc. are met.
- 4.1.10.14 Removal and reinstallation of threaded pipe caps or hose fittings downstream of drains, vents, etc.
- 4.1.10.15 Removal and reinstallation of blind flanges, gaskets, studs, and nuts on pipe downstream of drains and vents. (NON-Q SYSTEMS ONLY)
- 4.1.10.16 Performance of routine lubrication in accordance with Procedure 20411-C, "Control Of Lubricants".
- 4.1.10.17 Installation and removal of scaffolding in accordance with Procedures 20003-C, "Scaffold Construction And Control".

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- 4.1.10.18 Removal and reinstallation of insulation in accordance with Procedure 20002-C, "Control Of Insulation Removal And Installation".
- 4.1.10.19 Repairs on telecommunications equipment other than Security Communications equipment.
- 4.1.10.20 Adjustments of pump packing by qualified maintenance personnel (replacement of packing will require a WRT and MWO.)
- 4.1.10.21 Application of sealant to secondary equipment and piping to stop identified in-leakage.
- 4.1.10.22 Repairs to doors. (except fire doors)
- 4.1.10.23 Repair rework and adjustment of tooling that does not have a permanent plant tag number
- 4.1.10.24 Work on ancillary buildings such as the Service Building, Administration Building, etc.
- 4.1.11 Support Work Orders (SWO's) are required for the following:
 - 4.1.11.1 Facility work such as painting, labeling, work on handrails, lighting systems, etc.
 - 4.1.11.2 Spare parts rebuild.
 - 4.1.11.3 Repairs to fire doors.
- 4.1.12 MWOs generated per 4.1.11 will be reviewed as required in accordance with 29402-C.

NOTE

Support Work Orders (SWO's) may or may not require all review signatures. The Work Planner may obtain or N/A any review as determined appropriate.

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- 4.1.13 Maintenance will be scheduled and planned so as not to compromise the safety of the plant. Planning will consider the possible safety and ALARA consequences of concurrent or sequential maintenance, testing or operating activities. Equipment required to be operable for the prevailing mode will be available, and maintenance will be performed in a manner such that license limits are not violated. Planning for maintenance will include evaluation of the use of special processes, equipment, and materials in performance of the task, including assessment of potential hazards to personnel and equipment.
- 4.1.14 Work orders which modify the plant configuration will be processed under the controls of procedure 00400-C "Plant Design Control".
- 4.1.15 The WPG will assign the MWO package to the Supervisor/Foreman of the responsible group for implementation of the work package. The MWO will be packaged in accordance with Procedure 29402-C, "WPG Work Request Processing".
- 4.1.16 When personnel are ready to start work, the individual will go to the Control Room and obtain authorization to begin work from the applicable Shift Supervisor (SS), with the following exceptions:

NOTE

When investigative type WO work, not under a clearance, is continued beyond one shift the Unit Shift Supervisor shall be notified at the start of the new shift prior to continuing work, unless the USS indicates otherwise.

- 4.1.16.1 If a WO is security-related and DOES NOT require a clearance, the individual will go to the Supervisor Nuclear Security - Captain (SNS-CPT) to obtain authorization to begin work. The SNS-CPT or designee, will sign the WO to authorize work to begin.
- 4.1.16.2 If a WO is security-related and DOES require a clearance, the individual will go to the Shift Supervisor (SS) who will initial and date the WO and authorize work to begin, in accordance with step 4.1.16.

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- 4.1.16.3 If a WO is related to the demin water plant but does not affect the water-making capability of the plant, the individual will go to the Chemistry Foreman to obtain authorization to begin work.
- 4.1.16.4 If the Work Order does not affect plant operation or the load carrying capability of the plant, then the Operations Work Planning representative may approve the WO. These WO's may include but are not limited to; fire protection, coatings, structural, heat tracing, work performed in shops, and other general facility work.
- 4.1.17 Authorized Operations personnel will verify that the equipment or system can be released and determine what length of time it may be out of service.
- 4.1.18 Equipment clearance will be obtained, as necessary, from the SS per Procedure 00304-C, "Equipment Clearance And Tagging" prior to beginning work.
- 4.1.19 If the work task is in a closed tank, vessel, space, electrical manhole, or room with no ventilation, the individual doing the work, prior to beginning the work task, will contact appropriate personnel per Procedure 00258-C, "Safe Work Procedure For Closed Vessels, Confined Spaces, Wet Locations And Systems".
- 4.1.20 If the work task impacts a Fire Protection Program component such that a FP LCO will be entered, the SS will ensure that the required LCO actions are implemented prior to the existence of the limiting condition.
- 4.1.21 The individual foreman/supervisor will designate the appropriate Housekeeping Zone and/or cleanliness controls in accordance with Procedure 20427-C, "Maintenance Cleanliness And Housekeeping Control" and indicate the requirements on the WO Form.
- 4.1.22 Authorization to begin work will be documented on the MWO/SWO by the applicable Shift Supervisor, SNS-CPT Chemistry Foreman, or OPS WPG Rep (See 4.1.16) signing the WO and the Unit SS issuing a subclearance, as required, to the foreman/supervisor responsible for the work.

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- 4.1.23 If the work task requires draining of water in Secondary systems or subsystems, contact Chemistry Section personnel prior to draining. For Primary systems/subsystems contact Radwaste Section Personnel prior to draining.
- 4.1.24 The assigned work crew will:
 - 4.1.24.1 Obtain working copies of procedures as required for data sheets and/or procedures listed or referenced in WO package.
 - 4.1.24.2 Verify clearance boundaries. Notify Control Room when valves are repositioned during maintenance activities.
 - 4.1.24.3 Review the RWP, receive a pre-job or post-job radiological briefing (if required), sign in and out on the RWP, and notify HP if the work is suspended or terminated.
 - 4.1.24.4 Notify QC personnel, four hours in advance, if possible, to witness the predetermined step(s). Work is not to proceed beyond a QC inspection, witness or Hold Point unless a waiver has been approved by QC personnel. Contractor maintenance activities shall be worked to their specific QC/QA procedure where applicable.
 - 4.1.24.5 Observe safe, efficient, and professional work practices.
 - 4.1.24.6 Prior to working on equipment, check equipment ID number to be sure the correct equipment is being worked. System draining will be coordinated by Operations with the HP/Chemistry Department.
 - 4.1.24.7 Investigate and document the cause of the malfunction. Investigations, inspections, examinations and observations pertinent to evaluating the cause of the malfunction and/or corrective action will be documented on the WO.
 - 4.1.24.8 Attach all applicable documents to the WO package.
 - 4.1.24.9 Document, on the Work Order, all work performed during the work activity. Should additional space be required to properly document complex work performed, a Work Order Continuation Sheet (Figure 2) will be used. If more than one Continuation Sheet is used, each will be sequentially numbered (in upper-right corner).

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NOTE

If the maintenance required for a particular work order involves more than one crew, each foreman/supervisor will sign and date the WO, that their work is complete or as complete as possible (including specific steps of procedure completed) and has been reviewed and approved prior to the transfer of the WO to the next responsible crew.

- 4.1.24.10 Document the disposition of replaced equipment/parts and list both the old and new part numbers/serial numbers on the MWO.
- 4.1.24.11 If warehouse material was required to complete the work, a check mark will be placed in Block 28. Material/Equipment Request (MER) numbers will be recorded in the block. A copy of the MER will be attached to the Work Order (WO). Lot, reel, spool, heat, grade, part, serial numbers, etc., will be recorded on the WO or the attached MER. All material removed shall be documented on the work order with as much information on the item removed, as available.
- 4.1.24.12 The use of bulk materials, such as, thermal overloads, lugs, fittings, packing, etc., shall be controlled in accordance with Procedure 00352-C, "Control Of In-Process Materials", unless the quality and traceability of the material can be established by other means. The part number and MIR/MER number from the bulk material identification tag will be written on the WO.
- 4.1.25 If all the work required cannot be performed, the work foreman/supervisor will document, on the WO, why the work could not be completed and sign and date at the end of the description. Return the WO package to WPG for further evaluation and/or instructions.
 - a. The WPG will determine if the WO is to be revised due to a scope and/or intent change or if a new MWO is required.

- b. When a revision is required, a revision sheet will be initiated and routed for review in accordance with the instructions for WO revision sheet and Procedure 29402-C, "WPG Work Request Processing".
- c. The WO will then be returned to the field for work to be completed as required.

4.1.26 During maintenance activities on non-safety related critical components or safety-related components, if it is necessary to change the scope or intent of the work, a new WO will be generated or the original will be returned to the WPG for revision to encompass the changed scope or intent. WPG will determine if a new WO is to be generated or the original WO amended. The new or revised original WO will be processed in accordance with all previous steps of this procedure. If a new WO is issued, it will reference the old WO number.

4.1.27 During maintenance activities on non-safety related non critical components, the MWO may be revised by the foreman, as required, to accomplish the required maintenance. If this is required, the foreman should contact the SS, NSSS, or Chemistry Foreman (as outlined in Step 4.1.16) to inform him that additional work is required. The foreman will clearly document all additional work on the WO.

4.1.28 The job foreman, upon completion of work, will:

- 4.1.28.1 Review the applicable documentation for completeness and verify that acceptance criteria for the identified problem has been met if known.
- 4.1.28.2 Ensure that any unsatisfactory work results or conditions noted during the performance of the work have been documented on the WO.
- 4.1.28.3 Ensure that the WO is completed, as applicable per 29402-C.
- 4.1.28.4 Inspect and ensure the work area is returned to the level of cleanliness required for the area.
- 4.1.28.5 Ensure the "Work Request Tag" has been removed from the equipment, and attach the WRT to the Work Order (WO)

PROCEDURE NO	REVISION	PAGE NO
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- 4.1.28.6 Sign and date the appropriate block of the WO after insuring that all blocks that are not applicable are marked N/A.
- 4.1.28.7 Ensure that inspections of work are performed, as required.
- a. Inspections are performed during and/or after the work to ensure the quality of the work and compliance with design. Inspections may be identified and performed by qualified individuals as appropriate.
 - b. The results of all inspections performed will be documented. Further action based on results and the documentation will be part of the WO package. Such documentation may include:
 - (1) Quality Control Inspection Reports,
 - (2) Maintenance Procedure Technical Results,
 - (3) Instrument Control Calibrations, etc.
- 4.1.29 QC personnel will perform required inspections and verifications and document on the WO where appropriate.

NOTE

If work was performed and inspection under a contractor's GPC approved QA/QC Program, the contractor's QA representative will sign and date the WO where appropriate.

- 4.1.30 When the review is complete, the WO package will be returned to the WPG where a functional or special test requirement will be assigned, if required, using Procedure 29401-C, "Maintenance Work Order Functional Tests" as a guideline. After the Functional test is assigned the WO will be forwarded to the appropriate department to perform the Functional test.
- 4.1.30.1 If the WO is sent to any department, other than Operations, for the completion of the functional test requirements, that department will complete the functional test in coordination with the SS, or Chemistry, as necessary.

PROCEDURE NO	REVISION	PAGE NO
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- 4.1.30.2 If the functional test is acceptable, the person who performed the functional test will sign and date the WO and check the WO SAT and return the WO to WPG.
- 4.1.30.3 If not acceptable, indicate "UNSAT" on the WO and sign and date. State the reason for the UNSAT condition in the work performed section of the WO. Return the WO to the WPG for revision or generation of a new WRT and WO closure.

NOTES

- a. The work can continue on an UNSAT WO by use of a revision. A WO is not closed until accepted by the SS, SSS, or SNS-CPT (as appropriate).
 - b. Special tests on security systems will be performed by a Security Supervisor, or designee, to ensure regulatory requirements are maintained.
- 4.1.31 When the work task is complete, the WO package will be returned to the SS, SSS or SNS-CPT (as appropriate, see Step 4.1.16) for closeout as follows:
- 4.1.31.1 If a clearance was required, the equipment clearance, obtained per Procedure 00304-C, "Equipment Clearance And Tagging", will be closed out, if all sub-clearance holders are signed off.
- 4.1.31.2 The system/equipment should be restored to normal per the applicable operating procedure as determined by the SS or SSS.
- 4.1.31.3 Verifying that surveillance, functional, acceptance, inspections and/or special tests are completed, as required, and the WO is completed as applicable.

NOTE

Shield plugs and blockwalls, where required, shall be reinstalled after completion of the functional test or after verification that the functional test does not require access to the component.

- 4.1.31.4 The WO will be signed by the SS, SSS, or SNS-CPT (as appropriate - see Step 4.1.16), if acceptable. If not acceptable, the SS, SSS or SNS-CPT will sign, date, and write UNSAT on the WO.
- 4.1.31.5 The Fire Protection LCO action may be released provided the condition causing the LCO no longer exists.
- 4.1.31.6 All of the WO package will be returned to the WPG except Safeguard WOs, which will be returned to Document Control.
- 4.1.31.7 If related to a security system, ensure that the Security Supervisor has been notified.
- 4.1.32 WO's with partial work complete may be closed if remaining work to be completed is on equipment/items not requiring an WO or on open PM's verified not to have commitments. This only voids remaining work and the WO shall document the reason for not completing remaining tasks.

4.2 CALIBRATION PROGRAM

The preventive maintenance program will determine scope and schedule calibration of instrumentation not covered by Technical Specifications in accordance with Procedure 20015-C, "Prevent Maintenance".

- 4.2.1 Calibration frequencies will be maintained as part of the program.
- 4.2.2 Calibration schedule and frequencies may be altered as operating experience is gained and as equipment history is developed.

4.3 VOIDING WO's

- 4.3.1 All WO's to be voided will be returned to WPG, with proper documentation on the WO, for evaluation and voiding per Procedure 29402-C, "WPG Work Request Processing".
- 4.3.2 Open PM's verified not to have commitments may be voided with a statement on the WO noting no commitments identified.

4.4 EMERGENCY MAINTENANCE

- 4.4.1 Emergency maintenance may be performed without the issuance or approval of an WO or procedure.
- 4.4.2 Emergency maintenance will be documented and reviewed promptly after completion of the work, or as plant conditions permit by use of the WO.
- 4.4.3 Emergency maintenance will only be authorized by the General Manager - Nuclear Plant, Assistant General Manager - Plant Operations, OSOS, or their respective designee.
- 4.4.4 If time permits, an Emergency WO should be processed per Subsection 4.5.

4.5 EMERGENCY WORK ORDER PROCESSING

Under emergency operating conditions, as determined by the On-Shift Operations Supervisor, (OSOS), where immediate actions are required to protect the health and safety of the public and plant personnel, to protect equipment or prevent deterioration of plant conditions to an unsafe level, maintenance activities may be accomplished without the use of written procedures. After accomplishing the maintenance work, it shall be documented and given the same degree of review as though preplanned and performed according to written procedure. Emergency work will be performed as outlined.

- 4.5.1 If the need for an emergency WO arises, as determined by the OSOS or designee, the OSOS or designee will:
- 4.5.1.1 Notify on-shift maintenance supervision of the declaration of an emergency WO.

05-95-1-90

NP1111MG NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM 03/31/88
 NP+000DM MAINTENANCE WORK ORDER (1 OF 2) 10 24 00
 PRINT WALKDOWN RPT PRINTER ID NEXT PAGE 1
 FUNC I MWO# 18807746 00 UNTL# *1000AG REF TASK# PRI 34641
 DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE
 REF TASK# STATUS INIT XMIT NUMBER
 DATE LOADED 10/30/88 TIME LOADED 16 17 40 JOB NUMBER X EQ (Y/N)
 TYPE MAINT (P/F) P MWO SAFETY CLASS (S-SAFETY/N-NON-SAFETY) N
 MPL/TAG LIST TR IND DESC NUC CLS
 SYSTEM DESC NFRD VEND/MFG DATE CODE
 TECH SPECS

LOC PAID MECH
 ELEM ELEC CONN
 PROBLEM/WORK REQUESTED DESC **FOR INFORMATION**
 DURING DIESEL GEN. RUN RECIEVED ALARM 0077 - D/G HI JACKET WATER TEMP.
 MALFUNCTION*. LOCAL INDICATION SHOWED J.W. TEMPTS. NORMAL.
 DOES NOT AFFECT DIESEL OPERABILITY

SHOULD WORK WHEN 18807710 IS WORKED*(SEE PAGE 2 COMMENTS. SMP 9/13/89)

INITIATOR CHUCK LADD SUPERVISOR CHUCK LADD DATE REQD CONT
 WCC RECEIVED DATE 11/21/89 TEMP MODS
 EQP MODE REQD (O/F/N) REPAIR TAGS
 UNIT MODE REQD (1-6/O/N) NCR/DR # (Y/N) Y NCR/DR # 1883453
 FIRE PROTECT RELATED (Y/N) N SIGNATURE MATL AVAIL(Y/N) Y
 REJECT CD/NAME/DT DCR # (Y/N) N DCR #
 P2:START END OUTAGE (Y/N) Y FORCED REFUEL 12 REFERENCE 0

COMPLETE TRANSACTION

7-BK 8-FWD 11-TXT 14-ST HIST/LIST 15-EXIT 17-EQP DTL 18-QUIT 19-REPDTL

NP1112MG NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM 03/31/88
 NE1000DM MAINTENANCE WORK ORDER (2 OF 8) NEXT PAGE 2 10:24:00
 FUNC I MWO# 18907746 00 CNTL# *1000AG REP TASK# PRI 34641
 DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE
 SCAFFOLDING IND REQUEST # INSULATION IND REQUEST #
 OP X HP CH SE EL ME IC EN QC OT SCHED DT END
 LEAD DISC MAIN RESP FORE MWO CLEAR REQ'D(Y/N) Y # 18900758
 DPT RESP MAINT ICOP RESP SUPV EST WORK DURATN 12.0 PCT COMPLETE
 WELD PERMIT REQD(Y/N) # RWP REQD(Y/N) N # FUNC TEST(Y/N)
 MATL REQ'D (Y/N) Y COMMENTS GP25-902346 PD #7000195 MER 016194 (KIT)
 RESERVE # QC HOLD PTS (Y/N) Y QC REVIEW R L HEATER DATE 10/31/88
 CRAFT/COMPLETE MECH N ELEC N I&C Y HP N CONT N OTHER N TOTALS
 RESP FOREMAN
 DATE SCHED START
 DATE SCHED END

EST CREWSIZE			2						2
EST MANHOURS	.0	.0	24.0	.0	.0	.0	.0	.0	24.0
EST EXPOSURE									
ACT CREWSIZE			2						2
ACT MANHOURS	.0	.0	20.0	.0	.0	.0	.0	.0	20.0
ACT EXPOSURE									

PROC # 22332-C 20429-C LCO (Y/N) N
 WORK *
 INSTRUCT
 REWORK/REPLACE ANY OF THE SWITCHES AS REQUIRED TO CORRECT MALFUNCTION.

MAINTAIN ZONE IV HOUSEKEEPING

CONT

 COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF11-CONT TEXT PF14-RESERVE LIST PF15-EXIT PF18-QUIT

NF1150MG
NF1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER
CONTINUATION

03/31/89
10:24:00

COMMENT TYPE INSTRUCT PAGE 01
REV. 1 5/4/89 J S KITCHENS

BLK 23: ADD CALIBRATION OF TEMP SWITCHES PER ATTACHEMNT #1.

NEXT PAGE 01

PF15-EXIT PF18-RETURN TO MWO FILE MAINT ENTER NEXT PAGE OR PF18
PF7-BACK PF8-PAGE

NP1113MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (3 DE 8)

03/31/91
10 24 0

FUNC I MWO# 18907746 DO CNTRL# *1000AG REP TASK#
DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE
REF TASK#

NEXT PAGE 3

PRI 34641

INITIATION REVIEW

OPS JCP DATE 10/31/88 MNT TR DATE 10/31/88 HP HMB DATE 10/30/88
ENG MWP DATE 10/31/88 SMI/MER # 89014200 89014163 89014162
SPECIAL REVIEW REQD(Y/N) N SIGNATURE
SFC REV REQD COMM

DATE RELEASED FOR WORK 11/03/88 SIGNATURE W RYAN

DATE ACTUAL BEGIN 11/03/88 DATE ACTUAL END 11/20/89

DUE DATE EARLY START DATE LATEST END DATE

ACT WORK APPLIED 40-60 PSI TO 1-TSH-19110, 1-TSH-19112 THRU DRFFICE AND HAD NO
PERFORMED EAKS. DIDN'T USE PROCEDURES 20429-C/22332-C USED VP-2346 DUE 1-24-89
MAINTAINED ZONE IV HOUSEKEEPING ZONE IV HOUSEKEEPING WAS MAINTAINED
OBTAINED PERMISSION FROM SS (RYAN) TO BEGIN WORK, REMOVED 1TSH19110 &
1TSH19111, FROM SERVICE PROCEEDED TO I & C SHOP TO CALIBRATE ON BENCH SE
T-UP. OBTAINED AS-FOUND READINGS ON SWITCHES, BOTH 19110 & 19111 WERE
OUT OF AS LEFT TOLERANCES. BEFORE CALIBRATIONS COULD BE MADE OPERATION
S REQUESTED SWITCHS BE RE-INSTALLED. INSTALLED 1TSH19110 & 1TSH19111 O
N JACKET WATER HEADER OF DG 1A. USED PROC 22332-C TO TAKE AS FOUNDS,
FOREMEN

CONT

HIST SUMMARY

PERSON PERFORMING WORK TIM NEAL DATE COMPLETED 11/20/89

MAINT FOREMAN SCOTT HAMMOND REVIEW DATE 11/20/89

CORRECTIVE ACTION CODES

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF11-CONT TEXT PF15-EXIT PF18-QUIT

CONTINUATION

COMMENT TYPE PERFORMED PAGE 01
USED THE FOLLOWING TEST EQUIP VP2447-60#GAGE -1-24-89, VP3081
-FLUKE 2190A-1/26/89, VP2444-60# GAGE -1-9-89. BOTH SWITCHES
REFLECT CURRENT PLANT CONDITIONS. MAINTAINED ZONE IV HOUSEKE
EPING 110389 WTC PARTS 110388 PGL WROTE DC #1-88-3453

OBTAINED AN ORFICE UNDER MER #89-6323. DRILLED OUT ORFICE
WITH A #70 DRILL (VERIFIED WITH A MICROMETER TO BE .028).
VERIFIED THAT THE .028 DRILL WAS TIGHT IN THE HOLE. OBTAINED
THREE NEW CALCON TEMP. SWITCHES FROM N.O. WHSE UNDER MER#89-
6307. ATTEMPTED TO CALIBRATE SWITCHES USING ATTACHMENT #1.
TWO SWITCHES ARE DEFECTIVE. ONE SWITCH HAS 2 STRIPPED SCREWS
ON THE COVER AND THE OTHER LEAKS AIR AROUND THE COVER AND
AROUND THE INPUT AIR THREADS. THE THIRD SWITCH IS OUT OF
TOLERANCE (AS-FOUND OF 207 PSI), HOWEVER NO ADJUSTMENTS WERE
MADE YET. THE OUT-OF-TOLERANCE SWITCH IS IN "B" STORAGE.
GEORGE TECHENTINE 5/10/89

NEXT PAGE 01

FF15-EXIT PF18-RETURN TO MWO FILE MAINT ENTER NEXT PAGE OR PF18
PF7-BACK PF8-PAGE

NP1114MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (4 OF 8)

03/31/79
10:24

NEXT PAGE 4

FUNC I MWD# 18807746 00 CNTL# *1000AG REP TASK# FRI 34641
DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE

REF TASK#

TYPE OF FAILURE UNIT STATUS AT TIME OF FAILURE
CAUSE OF FAILURE METHOD OF DETECTION
MODE OF FAILURE CAUSE DESCRIPTION CODES
EFFECT ON PLANT EFFECT ON SYSTEM

INSPECTION PERFORMED BY DATE

POST MAINT FUNC TEST CODES

METHOD FUNC TEST NONE REQUIRED PERFORMED IN BLOCK 27

DATE FUNCTIONAL TEST ASSIGNED

DPT RESP FNIST

FUNC TEST PERFORMED BY

DATE

PROCEDURE#

PROVES OPERABILITY (Y/N)

METHOD USED TO PROVE OPERABILITY

SATISFACTORY (Y/N) Y IF UNSAT. CORRECTIVE ACTION

NEW MWD#

OPERATIONS ACCEPTANCE BY W B DIEHL

DATE 11/21/89

OSOS APPROVAL

DATE

PERFORMANCE TRACKING SCHD BEGIN DATE

REPORTED BEGIN DATE

SCHD END DATE

REPORTED END DATE

INITIALS

RCN

SPECIAL REVIEW COMPLETE (Y/N) PERFORMED DATE

MEETING#

SIGNATURE

COMMENTS

CLOSE OUT REVIEW BY QC J C HARVEY

QC CLOSE DATE 11/21/89

TRANSMITTAL NUMBER 107661

TRANSMITTAL DATE: 11/27/89

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF14-LIST PF15-EXIT PF18-QUIT

NP1115MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER

03/31/9
10 24 0

NEXT PAGE 5

FUNC I MWO# 18807746 00 CNTL# *1000AG REF TASK#
DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE
REF TASK#

MPL/TAG	TR	SYS	DESCRIPTION	LOCATION	EQP CL
1161905ALB035		1619	ANNUNCIATOR LIGHT BOX ALB	1CB163	62J
1TSH19110	A	2403	DG1 JACK WTR HDR OUT	1DB1	61J
1TSH19111	A	2403	DG1 JACK WTR MAIN HDR OUT	1DB1	61J
1TSH19112	A	2403	DSL GEN1 JACK WTR MAIN HD	1DB1	61J

CURRENT MPL/TAG PAGE 1 OF 1 NEXT MPL/TAG PAGE 1

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF10-MNTHIST PF14-LIST PF15-EXIT PF17-EQP DTL PF18-QUI

NP1119MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER

03/31/88
10 24

NEXT PAGE 8

PUNC I MWD# 18807746 00 SNTL# *1000AG REP TASK#
DATE 10/30/88 TIME UNIT 1 SYSTEM STATUS F WORK TYPE C SUB TYPE
REF TASK#

ACCOUNT NUMBER				ANALYSIS		PAC	
FE	LOCN	FERC SUB	WO	CODE	RCN	CODE	
				01			

COMPLETE TRANSACTION

PF7-BACK MWO PF8-FWD MWO PF14-LIST PF15-EXIT PF18-QUIT

05-95-2-90

NP1111MG NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM 03/31/90
 NP1000DM MAINTENANCE WORK ORDER (1 OF 8) 10:28:11
 PRINT WALKDOWN RPT PRINTER ID NEXT PAGE 1
 FLUNC 1 MWO# 19001339 00 CNTL# ** G REF TASK# PRI 34
 DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1F WORK TYPE C SUB TYPE
 REF TASK# STATUS: INIT XMIT NUMBER
 DATE LOADED 3/15/90 TIME LOADED 9:25:00 JOB NUMBER EQ (Y/N)
 TYPE MAINT (P/F) P MWO SAFETY CLASS (S-SAFETY/N-NON-SAFETY) N
 MPL/TAG LIST TR IND DESC NMC CLS
 SYSTEM DESC NFRD VEND/MFG DATE CODE
 TECH SPECS
 LOC P&ID MECH
 ELEM ELEC CONN
 PROBLEM/WORK REQUESTED DESC
 D/G 1A HIGH PRESSURE STARTING AIR ALARM IS LIT WITH RECEIVER @ 238/
 240 PSIG.
 LOCATION D/G 1A. QMCB ALARM

FOR INFORMATION

WRT 6154 CONT
 INITIATOR SN DYER SUPERVISOR GARY MOORE DATE REQD
 WCC RECEIVED DATE 3/25/90 TEMP MODS
 EQP MODE REQD (O/F/N) REPAIR TAGS 06154
 UNIT MODE REQD (1-6/O/N) NCR/DR # (Y/N) N NCR/DR #
 FIRE PROTECT-RELATED (Y/N) N SIGNATURE MATL AVAIL(Y/N)
 REJECT CD/NAME/DT DCR # (Y/N) N DCR #
 P2 START END OUTAGE (Y/N) Y FORCED REFUEL 12 REFERENCE D

COMPLETE TRANSACTION

7-BK 8-FWD 11-TXT 14-ST HIST/LIST 15-EXIT 17-EQP DTL 18-QUIT 19-REFDTL

NP1112MG

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM

03/31/90

NP1000DM

MAINTENANCE WORK ORDER (2 OF 8)

NEXT PAGE 2

10:28:2

FUNC I MWD# 19001337 00 CNTL# ** G REF TASK#

PRI 34

DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1/ WORK TYPE C SUB TYPE

SCAFFOLDING IND REQUEST INSULATION IND REQUEST #

OP HP CH SE EL ME IC EN QC OT SCHED DT 3/23/90 END 3/24/90

LEAD DISC MAIN FTSP FORE OPS CLEAR REQ'D(Y/N) N #

DPT RESP MAINT ICOP RESP SUPV EST WORK DURATN 3.0 PCT COMPLETE

WELD PERMIT REQ'D(Y/N) N # RWP REQ'D(Y/N) N # FUNC TEST(Y/N)

MATL REQ'D (Y/N) COMMENTS

RESERVE # QC HOLD PTS (Y/N) N QC REVIEW TW FREEMAN DATE 3/15/90

CRAFT/COMPLETE MECH N ELEC N I&C Y HP N CONT N OTHER N TOTALS

RESP FOREMAN TGG

DATE SCHED START

DATE SCHED END

FST CREWSIZE

2

EST MANHOURS .0 .0 6.0 .0 .0 .0 .0

EST EXPOSURE

ACT CREWSIZE

2

ACT MANHOURS .0 .0 3.0 .0 .0 .0 .0

ACT EXPOSURE

PROC # 22721-C

20429-C

LCD (Y/N) N

WORK *CRI REPORTABLE

INSTRUCT

INVESTIGATE/RECAL OR REPLACE PRESS. SWITCHES AS REQUIRED TO CLEAR

ALARM CONDITION. RBRGSE 3/15/90

ACCURACY PER: AX4AK01-564 MAINTAIN ZONE IV HOUS KEEPING

SETPOINT PER: 1X4AK01-52 RESET FIXED P: CX5DT101-40 & 38 CONT

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF11-CONT TEXT PF14-RESERVE LIST PF15-EXIT PF18-CUIT

NP1112MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM

03/31/90

MAINTENANCE WORK ORDER (2 OF 8)

NEXT PAGE 2

10:28:2

FRI 34

PURC I HWDB 19001339 00 CNTL# ** G REP TASK#

DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1F WORK TYPE C SUB TYPE

SCAFFOLDING IND REQUEST # INSULATION IND REQUEST #

OP HP CH SE EL ME IC EN GC OT SCHD DT 3/23/90 END 3/24/90

LEAD DISC MAIN REST FORE OPS CLEAR SLD D(Y/N) N #

DPT RESP MAINT ICOP RESP SUPV EST WORK DURATN 3.0 PCT COMPLETE

WELD PERMIT REQD(Y/N) N # RWP REQD(Y/N) N # FUNC TEST(Y/N)

MATL REQ'D (Y/N) COMMENTS

RESERVE # QC HOLD PTS (Y/N) N QC REVIEW TW FREEMAN DATE 3/15/90

CRAFT/COMPLETE MECH N ELEC N I&C Y HP N CONT N OTHER N TOTALS

RESP FOREMAN TGG

DATE SCHED START

DATE SCHED END

EST CREW SIZE 2

EST MANHOURS .0 .0 6.0 .0 .0 .0 .0

EST EXPOSURE

ACT CREW SIZE 2

ACT MANHOURS .0 .0 3.0 .0 .0 .0 .0

ACT EXPOSURE

PROC # 22721-C 20429-C LCO (Y/N) N

WORK *CRI REPORTABLE

INSTRUCT

INVESTIGATE/RECAL OR F H AL PRESS. SWITCHES AS REQUIRED TO CLEAR

ALARM CONDITION, RBROSE 3/15/90

ACCURACY FER: AX4AK01-564 MAINTAIN ZONE IV HOUSEKEEPING

SETPOINT FER: 1X4AK01-52 RESET FIXED FER: CX5DT101-40 & 38 CONT

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF11-CONT TEXT PF14-RESERVE LIST PF15-EXIT PF18-QUIT

NP1113MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (3 OF 8)

03/31/90
10 28 2

NEXT PAGE 3

PLANT 1 MWO# 19001339 00 CNTL# ** G REP TASK# FRI 34
DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1F WORK TYPE C SUB TYPE
REF TASK#

INITIATION REVIEW

UPS JCP DATE 3/16/90 MNT RIR DATE 3/18/90 HP HMB DAT: 3/15/90
ENG KEH DATE 3/15/90 SMI/MER #
SPECIAL REVIEW REQD(Y/N) N SIGNATURE
SPC REV REQD COMM
DATE RELEASED FOR WORK 3/22/90 SIGNATURE WB DIEHL
DATE ACTUAL BEGIN 3/22/90 DATE ACTUAL END 3/24/90
DUE DATE EARLY START DATE LATEST END DATE
ACT WORK OBTAINED SS PERMISSION TO CALIBRATE HIGH AIR PRESSURE ALARM SWITCHES
PERFORMED FOUND BOTH SWITCHES OUT OF CAL. CALIBRATED SWITCHES PER PROCEDURE 20429-0
21-C USING VP2585 C/D 6-20-90. ADJUSTED TO WITHIN LIMITS. PLACED SWITCHES
BACK IN SERVICE. PROCEDURE 20429-0 NOT USED ZONE IV HOUSEKEEPING
G MAINTAINED.

CONT

FOREMEN

HIST SUMMARY

PERSON PERFORMING WORK MICHAEL JOHNSON DATE COMPLETED 3/23/90
MAINT FOREMAN P LINEBARGER REVIEW DATE 3/24/90

CORRECTIVE ACTION CODES

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF11-CONT TEXT PF15-EXIT PF18-QUIT

NP1114MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (4 OF 8)

03/31/90
10:28:11

NEXT PAGE 4

FUNC 1 MW08 1900:330 00 CNTL# ** G REP TASK# PRI 34
DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1F WORK TYPE C SUB TYPE

REF TASK#

TYPE OF FAILURE UNIT STATUS AT TIME OF FAILURE
CAUSE OF FAILURE METHOD OF DETECTION
MODE OF FAILURE CAUSE DESCRIPTION CODES
EFFECT ON PLANT EFFECT ON SYSTEM

INSPECTION PERFORMED BY DATE

POST MAINT FUNC TEST CODES

METHOD FUNC TEST

DATE FUNCTIONAL TEST ASSIGNED

DPT RESP FNTST

FUNC TEST PERFORMED BY

DATE

PROCEDURE#

PROVES OPERABILITY (Y/N)

METHOD USED TO PROVE OPERABILITY

SATISFACTORY (Y/N)

IF UNSAT, CORRECTIVE ACTION

NEW MW0#

OPERATIONS ACCEPTANCE BY

DATE

OSOS APPROVAL

DATE

PERFORMANCE TRACKING SCHD BEGIN DATE

REPORTED BEGIN DATE

SCHD END DATE

REPORTED END DATE

INITIALS

RCN

SPECIAL REVIEW COMPLETE (Y/N)

PERFORMED DATE

MEETING#

SIGNATURE

COMMENTS

CLOSE OUT REVIEW BY QC

QC CLOSE DATE

TRANSMITTAL NUMBER

TRANSMITTAL DATE

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF14-LIST PF15-EXIT PF18-QUIT

NP1115MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (5 OF 8)

03/31/90
10 25 2

NEXT PAGE 5

FUNC I MW00 19001339 00 CNTLE ** C REP TASK# PRI 34
DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS IF WORK TYPE C SUB TYPE
REF TASK#
MPL/TAG TR SYS DESCRIPTION LOCATION EQP CL
1PSH9052 A 2403 STARTING AIR PRESS TRN A 1DB1 61J
1PSH9056 A 2403 STARTING AIR PRESS TRN A 1DB1 61J

CURRENT MPL/TAG PAGE 1 OF 1 NEXT MPL/TAG PAGE 1

COMPLETE TRANSACTION

PF7-BACK PF8-FWD PF10-MNTHIST PF14-LIST PF15-EXIT PF17-EQP DTL PF18-QUI

NP1119MG
NP1000DM

NUCLEAR PLANT MANAGEMENT INFORMATION SYSTEM
MAINTENANCE WORK ORDER (8 OF 8)

03/31/9
10:20:1

NEXT PAGE 8

FUNC 1 MWO# 19001339 00 CHTL# ** G REP ASK# PFI 34
DATE 3/12/90 TIME UNIT 1 SYSTEM STATUS 1F WORK TYPE C SUB TYPE
REF TASK#

ACCOUNT NUMBER					ANALYSIS	PAC	
FE	LOCN	FERC	SUB	WO	CODE	RCN	CODE
0000	009251	531	00027	000000	0100	000212	

COMPLETE TRANSACTION

PF7-BACK MWO PF8-FWD MWO PF14-LIST PF15-EXIT PF18-QUIT

OS-96-90

SECTION 6.0
ADMINISTRATIVE CONTROLS

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The General Manager - Nuclear Plant shall be responsible for overall plant operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The General Manager-Nuclear Plant will annually reissue a directive that emphasizes the primary management responsibility of the onshift Operations Supervisor (or during his absence from the control room, the individual designated to assume the command functions) for safe operation of the plant under all conditions on his shift and that clearly establishes his command duties.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATIONS

6.2.1 Onsite and offsite organizations shall be established for plant operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.
- b. The General Manager - Nuclear Plant shall be responsible for overall plant safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

PLANT STAFF

6.2.2 The plant organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

ADMINISTRATIVE CONTROLS

PLANT STAFF (Continued)

- b. When fuel is in either reactor at least one operator licensed on the applicable unit shall be in the control room. In addition, while either unit is in MODE 1, 2, 3, or 4 at least one Senior Operator licensed on the applicable unit(s) shall be in the control room.
- c. An individual* who has successfully completed the Initial Technician Training portion of the Health Physics Training Program or its equivalent shall be on site when fuel is in either reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. Administrative procedures shall be developed and implemented to limit the working hours of plant staff in performance of safety-related functions (e.g., licensed Senior Operators, licensed Operators, key Health Physics Technicians, key non-licensed operators, and key maintenance personnel).

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the plant is operating. (This work week may consist of 12-hour shift schedules.) However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification, on a temporary basis the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

#If a single Senior Operator does not hold a Senior Operator's license on both units, two or more Senior Operators who in combination are licensed as Senior Operators on both units may fulfill this requirement.

*This individual may be absent for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required position.

ADMINISTRATIVE CONTROLS

PLANT STAFF (Continued)

Any deviation from the above guidelines shall be authorized by the applicable department superintendent, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual excess overtime shall be reviewed monthly by the General Manager - Nuclear Plant or his designee to assure that excessive hours were authorized and that they do not become routine.

FIGURE 6.2-1
(DELETED)

FIGURE 6.2-2
(DELETED)

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION
TWO UNITS WITH A COMMON CONTROL ROOM

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION		
	BOTH UNITS IN MODE 1, 2, 3, or 4	BOTH UNITS IN MODE 5 or 6 OR DEFUELED	ONE UNIT IN MODE 1, 2, 3, or 4 AND ONE UNIT IN MODE 5 or 6 or DEFUELED
OS	1	1	1
SRO	1	none**	1
RO	3*	2*	3*
NLO	3*	3*	3*
STA	1***	none	1***

OS - Operations Supervisor with a Senior Operator license
SRO - Individual with a Senior Operator license
RO - Individual with an Operator license
NLO - Non-Licensed Operator
STA - Shift Technical Advisor

The shift crew composition may be on less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Operations Supervisor from the control room while either unit is in MODE 1, 2, 3, or 4, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Operations Supervisor from the control room while either unit is in MODE 5 or 6, an individual with a valid Senior Operator license or Operator license shall be designated to assume the control room command function.

*At least one of the required individuals must be assigned to the designated position for each unit.

**At least one licensed Senior Operator or Licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities must be present during CORE ALTERATIONS on either unit.

***The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Operations Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as stated in the Policy Statement on Engineering Expertise on Shift, dated October 28, 1985.

ADMINISTRATIVE CONROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of plant design and operating experience information, which may indicate areas for improving plant safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving plant safety to the Vice President-Nuclear.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers. Each shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his field, at least 1 year of which experience shall be in the nuclear field.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Vice President - Nuclear.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the plant for transients and accidents, and in plant design and layout, including the capabilities of instrumentation and controls in the control room.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.3 TRAINING

6.3.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Plant Training and Emergency Preparedness Manager. Personnel will meet the minimum education and experience recommendations of Regulatory Guide 1.8, Revision 2 and, for licensed staff, 10 CFR 55.59 before they are considered qualified to perform all duties independently. Prior to meeting the recommendations of Regulatory Guide 1.8, Revision 2, personnel may be trained to perform specific tasks and will be qualified to perform those tasks independently. Personnel who complete an accredited program which has been endorsed by the NRC shall meet the requirements of the accredited program in lieu of the above.

6.4 REVIEW AND AUDIT

6.4.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.4.1.1 The PRB shall function to advise the General Manager-Nuclear Plant on all matters related to nuclear safety.

COMPOSITION

6.4.1.2 The PRB shall be composed of Department Superintendents or Managers, or supervisory personnel reporting directly to Department Superintendents or Managers from the departments listed below:

- Operations
- Maintenance
- Quality Control
- Health Physics
- Nuclear Safety and Compliance
- Engineering Support

A senior health physicist is acceptable for the Health Physics Department PRB representative. The chairman, his alternate and other members and their alternates of the PRB shall be designated by the General Manager-Nuclear Plant.

ALTERNATES

6.4.1.3 No more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.4.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

QUORUM

6.4.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.4.1.6 The PRB shall be responsible for:

- a. Review of 1) procedures which establish plant-wide administrative controls to implement the QA program of Technical Specifications surveillance program, 2) procedures for changing plant operating modes, 3) emergency and abnormal operating procedures, 4) procedures for effluent releases of radiological consequences, and 5) fuel handling procedures.
- b. Review of 1) programs required by Specification 6.7.4 and changes thereto, and 2) proposed procedures and changes to procedures which involve an unreviewed safety question as per 10 CFR 50.59.
- c. Review of all proposed tests and experiments that affect nuclear safety;
- d. Review of all proposed changes to the Technical Specifications;
- e. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety, including proposed changes to Chapter 16.3 of the Vogtle Final Safety Analysis Report (FSAR);
- f. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President-Nuclear and to the Safety Review Board;
- g. Review of all REPORTABLE EVENTS;
- h. Review of plant operations to detect potential hazards to nuclear safety;
- i. Performance of special reviews, investigations, or analyses and reports thereon as requested by the General Manager-Nuclear Plant or the Safety Review Board;
- j. Review of the Security Plan and implementing procedures and submittal of recommended changes to the General Manager-Nuclear Plant and the Safety Review Board;

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager-Nuclear Plant and the Safety Review Board;
- l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear and to the Safety Review Board;
- m. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment Systems; and
- n. Review of the Fire Protection Program and Implementing procedures and submittal of recommended changes to the General Manager-Nuclear Plant.

6.4.1.7 The PRB shall:

- a. Recommend in writing to the General Manager-Nuclear Plant approval or disapproval of items considered under Specification 6.4.1.6a. through e. prior to their implementation;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.4.1.6a. through f. constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Vice President-Nuclear and the Safety Review Board of disagreement between the PRB and the General Manager-Nuclear Plant; however, the General Manager-Nuclear Plant shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.4.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President-Nuclear and the Safety Review Board.

6.4.2 SAFETY REVIEW BOARD (SRB)

FUNCTION

6.4.2.1 The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The SRB shall report to and advise the Vice President-Nuclear on those areas of responsibility specified in Specifications 6.4.2.7 and 6.4.2.8.

COMPOSITION

6.4.2.2 The SRB shall be composed of a minimum of five persons who, as a group, provide the expertise to review and audit the operation of a nuclear power plant. The chairman and other members shall be appointed by the Vice President-Nuclear or other such person as he may designate. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.4.2.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time.

CONSULTANTS

6.4.2.4 Consultants shall be utilized as determined by the SRB chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.4.2.5 The SRB shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.4.2.6 The quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the SRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

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REVIEW

6.4.2.7 The SRB shall be responsible for the review of:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or these Operating Licenses;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the PRB.

AUDITS

6.4.2.8 Audits of plant activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25% of the inspection audit interval.
2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;

ADMINISTRATIVE CONTROLS

AUDITS (continued)

- b. The performance, training, and qualifications of the entire plant staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;
- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used as least every third year;
- g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;
- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;
- j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;
- k. The Emergency Plan and implementing procedures (at least once per 12 months);
- l. The Security Plan and implementing procedures (at least once per 12 months).

RECORDS

6.4.2.9 Records of SRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved, and forwarded to the Vice President-Nuclear within 14 days following each meeting;

ADMINISTRATIVE CONTROLS

RECORDS (Continued)

- b. Reports of reviews encompassed by Specification 6.4.2.7 shall be prepared, approved, and forwarded to the Vice President-Nuclear within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.4.2.8 shall be forwarded to the Vice President-Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5 REPORTABLE EVENT ACTION

6.5.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Section 50.72 and Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PRB, and the results of this review shall be submitted to the SRB and the Vice President-Nuclear.

6.6 SAFETY LIMIT VIOLATION

6.6.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. In accordance with 10 CFR 50.72, the NRC Operations Center shall be notified by telephone as soon as practical and in all cases within one hour after the violation has been determined. The Vice President-Nuclear, the SRB, PRB, and the General Manager-Nuclear Plant shall be notified within 24 hours.
- b. A Licensee Event Report shall be prepared in accordance with 10 CFR 50.73.
- c. The Licensee Event Report shall be submitted to the Commission in accordance with 10 CFR 50.73, and to the PRB, SRB, the General Manager-Nuclear Plant and the Vice President-Nuclear within 30 days after discovery of the event and the covery of the event.
- d. Critical operation of the affected unit shall not be resumed until authorized by the Nuclear Regulatory Commission.

6.7 PROCEDURES AND PROGRAMS

6.7.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;

A. ADMINISTRATIVE CONTROLS

6.7 PROCEDURES AND PROGRAMS (Continued)

- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation;
- g. Quality Assurance for effluent and environmental monitoring;
- h. Fire Protection Program Implementation; and
- i. Technical Specifications Improvement Program implementation. (FSAR Chapter 16.3)

6.7.2 Each procedure of 6.7.1 above, and changes thereto, shall be reviewed as set forth in administrative procedures and approved by either the General Manager-Nuclear Plant or the department head of the responsible department prior to implementation with the exception of the following which shall be approved by the General Manager-Nuclear Plant:

- 1) procedures which establish plant-wide administrative controls (which implement the quality assurance program and the Technical Specifications surveillance program),
- 2) unit operating procedures (UOPs)
- 3) emergency operating procedures (EOPs)
- 4) abnormal operating procedures (AOPs)
- 5) procedures for implementing the security plan, emergency plan, and the fire protection program, and
- 6) fuel handling procedures.

PRB responsibilities for procedures are delineated in 6.4.1.

6.7.3 Temporary changes to procedures of Specification 6.7.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license; and

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- c. The change is documented, reviewed in accordance with Specification 6.7.2 and approved by the General Manager-Nuclear Plant or department head of the responsible department within 14 days of implementation.

6.7.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the following:

- 1) Residual Heat Removal System
- 2) Containment Spray System (excluding NaOH Subsystem)
- 3) Safety Injection (excluding Boron Injection & Accumulators)
- 4) Chemical and Volume Control System (Letdown, Boron Recycle, and Charging Pumps)
- 5) Post Accident Processing System
- 6) Gaseous Waste Processing System
- 7) Nuclear Sampling System (Pressurizer steam and liquid sample lines, Reactor Coolant sample lines, RHR sample lines, CVCS Demineralizer and Letdown Heat Exchanger sample lines only)

The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points,
- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective actions for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Post-Accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.

e. A program which will ensure the capability to monitor plant variables and systems operating status during and following an accident. This program shall include those instruments provided to indicate system operating status and furnish information regarding the release of radioactive materials (Category 2 and 3 instrumentation as defined in Regulatory Guide 1.97 Revision 2) and provide the following:

- 1) preventive maintenance and periodic surveillance of instrumentation.
- 2) pre-planned operating procedures and back-up instrumentation to be used if one or more monitoring instruments become inoperable.
- 3) administrative procedures for returning inoperable instruments to OPERABLE status as soon as practicable.

ADMINISTRATIVE CONTROLS

6.8 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.8.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Regional Administrator of the Regional Office of the NRC unless otherwise noted.

STARTUP REPORT

6.8.1.1 A summary report of plant startup and power escalation testing shall be submitted following: (1) receipt of an Operating License, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of either unit.

The initial Startup Report shall address each of the startup tests identified in the Final Safety Analysis Report and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report. Subsequent Startup Reports shall address startup tests that are necessary to demonstrate the acceptability of changes and/or modifications.

Startup Reports shall be submitted within: (1) 90 days following completion of the Startup Test Program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of Startup Test Program, and resumption or commencement of commercial operation), supplementary reports shall be submitted at least every 3 months until all three events have been completed.

ANNUAL REPORTS**

6.8.1.2 Annual Reports covering the activities of the plant as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of plant, utility, and other personnel (including contractors) receiving exposures greater than 100 mre.a/yr and their associated man-rem exposure according to work and job functions* (e.g., reactor operations and

*This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

**A single submittal may be made for Units 1 and 2. The submittal should combine those sections that are common to both units at the plant.

ADMINISTRATIVE CONTROLS

ANNUAL REPORTS (Continued)

surveillance, inservice inspection, routine maintenance, special maintenance [describe maintenance], waste processing, and refueling). The dose assignments to various duty functions may be estimated based on pocket dosimeter, thermoluminescent dosimeter (TLD), or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole-body dose received from external sources should be assigned to specific major work functions;

- b. The results of specific activity analyses in which the primary coolant exceeded the limits of Specification 3.4.8. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded (in graphic and tabular format); (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Clean-up flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration ($\mu\text{Ci/gm}$) and one other radioiodine isotope concentration ($\mu\text{Ci/gm}$) as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.
- c. A report shall be prepared and submitted to the commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of greater than or equal to 0.005 microcuries of removable contamination.

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT***

6.8.1.3 Routine Annual Radiological Environmental Surveillance Reports covering activities of the Radiological Environmental Monitoring Program during the previous calendar year shall be submitted prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following initial criticality and shall include copies of reports of the preoperational Radiological Environmental Monitoring Program of the plant for at least two years prior to initial criticality.

The Annual Radiological Environmental Surveillance Report shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including, as appropriate, a comparison with preoperational studies, with operational controls, and with previous environmental surveillance reports, and an assessment of any observed impacts of plant operations on the environment. The report shall also include the results of the Land Use Census required by Specification 3.12.2.

***A single submittal may be made for Units 1 and 2.

ADMINISTRATIVE CONTROLS

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT (Continued)

The Annual Radiological Environmental Surveillance Report shall include the results of analysis of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the table and figures in the Offsite Dose Calculation Manual, as well as summarized and tabulated results of these analyses and measurements in the format of the table in the Radiological Assessment Branch Technical Position, Revision 1, November 1979. The radiological level of radionuclides which are naturally occurring not included in the plant effluents need not be reported. In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as practicable in a supplementary report.

The report shall also include the following: a summary description of the Radiological Environmental Monitoring Program; at least two legible maps covering all sampling locations keyed to a table giving distances and directions from a point midway between the two reactors; the results of licensee participation in the Interlaboratory Comparison Program and the corrective action taken if the specified program is not being performed as required by Specification 3.12.3; reasons for not conducting the Radiological Environmental Monitoring Program as required by specification 3.12.1, and discussion of all deviations from the sampling schedule of Table 3.12-1; discussion of environmental sample measurements that exceed the reporting levels of Table 3.12-2 but are not the result of plant effluents, pursuant to ACTION b. of Specification 3.12.1; and discussion of all analyses in which the LLD required by Table 4.12-1 was not achieved.

SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

6.3.1.4 Routine Semiannual Radioactive Effluent Release Reports covering the operation of the unit during the previous 6 months of operation shall be submitted within 60 days after January 1 and July 1 of each year. The period of the first report shall begin with the date of initial criticality.

The Semiannual Radioactive Effluent Release Reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof. For solid wastes, the format for Table 3 in Appendix B shall be supplemented with three additional categories: class of solid wastes (as defined by 10 CFR Part 61), type of container (e.g., LSA, Type A, Type B, Large Quantity) and SOLIDIFICATION agent or absorbent (e.g., cement, urea formaldehyde).

The Semiannual Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year shall include an annual summary of hourly

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SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT (Continued)

meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing on magnetic tape of wind speed, wind direction, atmospheric stability, and precipitation (if measured), or in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability.* This same report shall include an assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from each unit during the previous calendar year. This same report shall also include an assessment of the radiation doses from radioactive liquid and gaseous effluents to MEMBERS OF THE PUBLIC due to their activities inside the SITE BOUNDARY (Figure 5.1-1) during the report period. All assumptions used in making these assessments, i.e., specific activity, exposure time, and location, shall be included in these reports. Historical annual average meteorological conditions or the meteorological conditions concurrent with the time of release of radioactive materials in gaseous effluents, as determined by sampling frequency and measurement, shall be used for determining the gaseous pathway doses. The assessment of radiation doses shall be performed in accordance with the methodology and parameters in the OFFSITE DOSE CALCULATION MANUAL (ODCM).

The Semiannual Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year shall also include an assessment of radiation doses to the likely most exposed MEMBER OF THE PUBLIC from reactor releases and other uranium fuel cycle sources within 8 km, including doses from primary effluent pathways and direct radiation, for the previous calendar year to show conformance with 40 CFR Part 190, "Environmental Radiation Protection Standards for Nuclear Power Operation." Acceptable methods for calculating the dose contribution from liquid and gaseous effluents are given in Regulatory Guide 1.109, Rev. 1, October 1977.

The Semiannual Radioactive Effluent Release Reports shall include a list and description of unplanned releases from the site to UNRESTRICTED AREAS of radioactive materials in gaseous and liquid effluents made during the reporting period.

The Semiannual Radioactive Effluent Release Reports shall include any changes made during the reporting period to the PROCESS CONTROL PROGRAM (PCP) and to the OFFSITE DOSE CALCULATION MANUAL (ODCM), pursuant to Specifications 6.12 and 6.13, respectively, as well as any major change to Liquid, Gaseous, or Solid Radwaste Treatment Systems pursuant to Specification 6.14. It shall also include a listing of new locations for dose calculations and/or environmental monitoring identified by the Land Use Census pursuant to Specification 3.12.2.

*In lieu of submission with the Semiannual Radioactive Effluent Release Report, the licensee has the option of retaining this summary of required meteorological data on site in a file that shall be provided to the NRC upon request.

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SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT (Continued)

The Semiannual Radioactive Effluent Release Reports shall also include the following: an explanation as to why the inoperability of liquid or gaseous effluent monitoring instrumentation was not corrected within the time specified in specification 3.3.3.9 or 3.3.3.10, respectively; and description of the events leading to liquid holdup tanks or gas storage tanks exceeding the limits of Specification 3.11.1.4 or 3.11.2.6, respectively.

OPERATING REPORTS

Monthly reports of operating statistics and shutdown experience, documentation of all challenges to the PORVs or safety valves, shall be submitted on a monthly basis to the Director, Office of Resource Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, no later than the 15th of each month following the calendar month covered by the report.

RADIAL PEAKING FACTOR LIMIT REPORT

6.8.1.6 The F_{xy} limits for RATED THERMAL POWER (F_{xy}^{RTF}) shall be established for at least each reload core and shall be maintained available in the Control Room. The limits shall be established and implemented on a time scale consistent with normal procedural changes.

The analytical methods used to generate the F_{xy} limits shall be those previously reviewed and approved by the NRC*. If changes to these methods are deemed necessary they will be evaluated in accordance with 10 CFR 50.59 and submitted to the NRC for review and approval prior to their use if the change is determined to involve an unreviewed safety question or if such a change would require amendment of previously submitted documentation.

A report containing the F_{xy} limits for all core planes containing Bank "D" control rods and all unrodded core planes along with the plot of predicted $F_{Q-P_{rel}}^T$ vs axial core height (with the limit envelope for comparison) shall be provided to the NRC Document Control desk with copies to the Regional Administrator and the Resident Inspector within 30 days of their implementation.

SPECIAL REPORTS

6.8.2 Special reports shall be submitted to the Regional Administrator of the Regional Office of the NRC within the time period specified for each report.

*WCAP 8385 "Power Distribution Control and Load Following Procedures" and WCAP 8272 "Westinghouse Reload Safety Evaluation Methodology."

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6.9 RECORD RETENTION

6.9.1 In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.9.2 The following records shall be retained for at least 5 years:

- a. Records and logs of plant operation covering time interval at each power level;
- b. Records and logs of principal maintenance activities, inspections, repair, and replacement of principal items of equipment related to nuclear safety;
- c. All REPORTABLE EVENTS;
- d. Records of surveillance activities, inspections, and calibrations required by these Technical Specifications;
- e. Records of changes made to the procedures required by Specification 6.7.1;
- f. Records of radioactive shipments;
- g. Records of sealed source and fission detector leak tests and results; and
- h. Records of annual physical inventory of all sealed source material of record.

6.9.3 The following records shall be retained for the duration of the plant Operating License:

- a. Records and drawing changes reflecting plant design modifications made to systems and equipment described in the Final Safety Analysis Report;
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories;
- c. Records of radiation exposure for all individuals entering radiation control areas;
- d. Records of gaseous and liquid radioactive material released to the environs;
- e. Records of transient or operational cycles for those plant components identified in Table 5.7-1;
- f. Records of reactor tests and experiments;
- g. Records of training and qualification for current members of the plant staff;

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of quality assurance activities required by the Final Safety Analysis Report;
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- k. Records of meetings of the PRB and the SRB;
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records;
- m. Records of secondary water sampling and water quality; and
- n. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.

6.10 RADIATION PROTECTION PROGRAM

6.10.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.11 HIGH RADIATION AREA

6.11.1 Pursuant to paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by paragraph 20.203(c), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates greater than 100 mrem/hr but less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area; or

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6.11 HIGH RADIATION AREA (Continued)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Health Physics Superintendent in the RWP.

6.11.2 In addition to the requirements of Specification 6.11.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the shift Foreman on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.12 PROCESS CONTROL PROGRAM (PCP)

6.12.1 The PCP shall be approved by the Commission prior to implementation.

6.12.2 Licensee-initiated changes to the PCP:

- a. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - 1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 - 2) A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and

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6.12 PROCESS CONTROL PROGRAM (PCP) (Continued)

- 3) Documentation of the fact that the change has been reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager-Nuclear Plant.

6.13 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.13.1 The ODCM shall be approved by the Commission prior to implementation.

6.13.2 Licensee-initiated changes to the ODCM:

- a. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - 1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered, dated and containing the revision number, together with appropriate analyses or evaluations justifying the change(s);
 - 2) A determination that the change will not reduce the accuracy or reliability of dose calculations or Setpoint determinations; and
 - 3) Documentation of the fact that the change has been reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager-Nuclear Plant.

6.14 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS*

6.14.1 Licensee-initiated major changes to the Radwaste Treatment Systems (liquid, gaseous, and solid):

- a. Shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the PRB. The discussion of each change shall contain:
 - 1) A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
 - 2) Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;

*Licensees may choose to submit the information called for in this Specification as part of the annual FSAR update.

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6.14 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS (Continued)

- 3) A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems;
 - 4) An evaluation of the change, which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments thereto;
 - 5) An evaluation of the change, which shows the expected maximum exposures to a MEMBER OF THE PUBLIC in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments thereto;
 - 6) A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period prior to when the change is to be made;
 - 7) An estimate of the exposure to plant operating personnel as a result of the change; and
 - 8) Documentation of the fact that the change was reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager-Nuclear Plant.