

# UNITED STATES

WASHINGTON, D.C. 20555-0001

#### VIRGINIA ELECTRIC AND POWER COMPANY

#### DOCKET NO. 50-280

#### SURRY POWER STATION, UNIT NO. 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 205 License No. DPR-32

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 10, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9509200057 950914 PDR ADOCK 05000280 P PDR

950

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:
  - (B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 205, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

non

David B. Matthews, Director Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 14, 1995

- 2 -



### UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

#### VIRGINIA ELECTRIC AND POWER COMPANY

#### DOCKET NO. 50-281

#### SURRY POWER STATION, UNIT NO. 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 205 License No. DPR-37

- The Nuclear Regulatory Commission (the Commission) has found that: 1.
  - The application for amendment by Virginia Electric and Power A. Company (the licensee) dated November 10, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - The facility will operate in conformity with the application, the Β. provisions of the Act, and the rules and regulations of the Commission:
  - There is reasonable assurance (i) that the activities authorized С. by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - The issuance of this amendment will not be inimical to the common D. defense and security or to the health and safety of the public; and
  - The issuance of this amendment is in accordance with 10 CFR Part Ε. 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:
  - (B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 205, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

luz

David B. Matthews, Director Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 14, 1995

#### ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 205 TO FACILITY OPERATING LICENSE NO. DPR-32 AMENDMENT NO. 205 TO FACILITY OPERATING LICENSE NO. DPR-37 DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages	Insert Pages	
TS 4.1-7	TS 4.1-7	
TS 4.1-8 TS 4.1-8a	TS 4.1-8 TS 4.1-8a	

#### TABLE 4.1-1 (Continued)

## MINIMUM FREQUENCIES STA CHECK, CALIBRATIONS, AND TEST OF INSTRUMENT CHANNELS

	Channel Description	Check	Calibrate	Test	Remarks
10.	Rod Position Bank Counters	S(1,2) Q(3)	N.A.	N.A.	<ol> <li>Each six inches of rod motion when data logger is out of service</li> <li>With analog rod position</li> <li>For the control banks, the bench- board indicators shall be checked against the output of the bank overlap unit.</li> </ol>
11.	Steam Generator Level	S	R	м	
12.	Charning Flow	N.A.	R	N.A.	
13.	Residual Heat Removal Pump Flow	N.A.	R	N.A.	
14.	Boric Acid Tank Level	*D	R	N.A.	
15.	Recirculation Mode Transfer a. Refueling Water Storage Tank Level-Low	S	R	м	
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	м	
16.	Volume Control Tank Level	N.A.	R	N.A.	
17.	Reactor Containment Pressure-CLS	*D	R	M(1)	1) Isolation valve signal and spray signal
18.	Boric Acid Control	N.A.	R	N.A.	
19.	Item Deleted				
20.	Deleted				
21.	Containment Pressure-Vacuum Pump System	S	R	N.A.	
22.	Steam Line Pressure	S	R	м	

TS 4.1-7

### TABLE 4.1-1 (Continued)

# MINIMUM FREQUENCIES FOR CHECK, CALIBRATIONS, AND TEST OF INSTRUMENT CHANNELS

	Channel Description	Check	Calibrate	Test	Bemarks
23.	Turbine First Stage Pressure	s	R	м	
24.	Ence gency Plan Radiation Instr.	"М	R	м	
25.	Environmental Radiation Monitors	•м	N.A.	N.A.	TLD Dosimeters
26.	Logic Channel Testing	N.A.	N.A.	M(1)(2)	1) Reactor protection, safety injection
27.	Turbine Overspeed Protection Trip Channel (Electrical)	N.A.	R	R	<ul> <li>and the consequence limiting safeguards system logic channels are tested monthly per this line kern.</li> <li>2) The master and slave relays are not included in the monthly logic channel test of the safety injection system.</li> </ul>
28.	Turbine Trip				Setpoint verification is not applicable
	A. Stop valve closure	N.A.	N.A.	Р	
	B. Low fluid oil pressure	N.A.	N.A.	Р	
29.	Seismic Instrumentation	м	R	м	
<b>3</b> Ú.	Reactor Trip Breaker	N.A.	N.A.	м	The test shall independently verify operability of the undervoltage and shunt trip attachments
31.	Reactor Coolant Pressure (Low)	N.A.	R	N.A.	

### TABLE 4.1-1 (Continued)

# MINIMUM FREQUENCIES FOR CHECK, CALIBRATIONS, AND TEST OF INSTRUMENT CHANNELS

	Channel Description	Check	Calibrate	Test	Remarks
32.	Auxiliary Feodwater				
	a. Steam Generator Water Level Low-Low	S	R	M(1)	1) The auto start of the turbine driven pump is not included in the monthly test, but is tested
	b. RCP Undervoltage	S	R	N.A.(1)	<ul> <li>within 30 days prior to each startup.</li> <li>1) The actuation logic and relays are tested</li> </ul>
	c. S.I.	(All Safety	Injection surveilla	Ince requirement	within 30 days prior to each startup.
	d. Station Blackout	N.A.	R	N.A.	
	e. Main Feedwater Pump Trip	N.A.	N.A.	R	
33.	Loss of Power				
	a. 4.16 KV Emergency Bus Under- vc?age (Loss of Voltage)	N.A.	R	м	
	<li>b. 4.16 KV Emergency Bus Under- voltage (Degraded Voltage)</li>	N.A.	R	м	
34.	Deleted				
35.	Manual Reactor Trip	N.A.	N.A.	R	The test shall independently verify the operability of the undervoltage and shunt trip attachments for the manual reactor trip function. The test shall also that the operability of the bypass breaker trip circuit.
36.	Reactor Trip Bypass Breaker	N.A.	N.A.	M(1), R(2)	<ol> <li>Remote manual undervoltage trip prior to placing breaker in service.</li> <li>Automatic undervoltage trip.</li> </ol>
37.	Safety Injection Input to RPS	N.A.	N.A.	R	
39.	Reactor Coolant Pump Breaker Position Trip	N.A.	N.A.	R	

TS 4.1-8a