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Millstone Nuclear Power Station
Northeast Nuclear Energy Company
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The Northeast Utilities System
Donald B. Miller Jr.,
Senior Vice President - Millstone

Re: 10CFR50.73(a)(2)(i)

September 14, 1995 MP-95-286

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Reference:

Facility Operating License No. DPR-65

Docket No. 50-336

Licensee Event Report 95-026-01

This letter forwards update Licensee Event Report 95-026-01.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Donald B. Miller, Jr.

Senior Vice President - Millstone Station

DBM/BW:ljs

Attachment: LER 95-026-01

cc: T. T. Martin, Region I Administrator

P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2, and 3

G. S. Vissing, NRC Project Manager, Millstone Unit No. 2

Test.

NRC Form 366 (5-92)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150 - 0104 EXPIRES: 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MINBE 7714). U.S. NUCLEAR REGULATORY COMMISSION. WASHINGTON, DC 20868-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET. WASHINGTON, DC 20603.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

DOCKET NUMBER (2)	PAGE (3)		
05000336	1 OF 2		

EVENT DATE (5) LER NUMBER (6)					3 (6)	REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)				
MONTH DAY		YEAR	YEAR	SEQUENTIAL NUMBER	REVISION		DAY YEAR		FACILITY NAME	DC	05000		
06	20	95	95	- 026 -	01	09	14	95	FACILITY NAME	DOCKET NUMBER 05000			
PERATI	NG		THIS	REPORT IS BEI	NG SUBM	ITTED PI	URSUA	OT TA	THE REQUIREMENTS OF 10 CFR	§: (Ch	eck one or more) (11)		
MODE (9)		5	20	0.402(b)		20.405(c)			50.73(a)(2)(h/)		73.71 (b)		
POWE	1		20	0.406(a)(1)(f)		50.36(c)(1)		50.73(a)(2)(v)		73.71 (c)		
LEVEL (10)		0	20	0.405(E)(1)(f)		50.36(c)(20		50.73(a)(2)(vii)		OTHER		
			25	0.405(a)(1)(H)	X	50.73(a)(2)(1)		50.73(a)(2)(MII)(A)	(Specify in Abstract below and in Text, NRC			
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			24	0.405(a)(1)(v)		50.73(a) (2) (10)		50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME

Philip J. Lutzi, Nuclear Licensing

TELEPHONE NUMBER (Include Area Code)

(203) 440-2072

CAUSE SYST	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFAC	TURER	11.0440.000	PRDS
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SUPPLEMENTAL REPORT EXPECTED (14)						F	KPECTED	МОМТН	DAY	YEAR		

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single - spaced typewritten lines) (16)

At 0158 on June 20, 1995, with the plant in Mode 5 at 0% power it was discovered that the double door assembly separating the railway access area from the 14'6" elevation general floor area of the auxiliary building was not correctly identified as a fire door required by Technical Specification 3.7.10 "Penetration Fire Barriers."

An assessment was made which concluded that the subject fire doors 205-14-007 were required to be maintained in accordance with the requirements of Technical Specification 3.7.10.

The root cause of the event was administrative personnel error.

Corrective action was to inspect the door and add it to the surveillance procedure.

This event is being reported pursuant to 10CFR50.73(a)(2)(i)(B), any operation or condition prohibited by the plant's Technical Specifications.

MR3 Form 3864 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MN88 7714). U.S. NUCLEAR REGULATORY COMMISSION. WASHINGTON, DC 20856-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20803.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)		
		YEAR		EQUENTA NUMBER		REVISION NUMBER				
Millstone Nuclear Power Station Unit 2	05000336	95	_	026	_	01	02	OF	02	

TEXT (If more apace is required, use additional copies of NRC Form 366A) (17)

Description of Event

At 0158 on June 20, 1995, with the plant in Mode 5 at 0% power it was discovered that the double door assembly separating the railway access area from the 14'6' elevation general floor area of the auxiliary building was not correctly identified as a fire door as required by Technical Specification 3.7.10 "Penetration Fire Barriers."

Although the door did have a "Fire Door" sign attached, it had not been previously identified properly as a Technical Specification door. Investigation revealed that the door had been omitted from the surveillance requirements of Technical Specification 3.7.10 and no record of surveillance could be found for a period which exceeded the frequency required.

All door assembly components critical to the fire resistance rating are UL listed. A review of all applicable work orders, in conjunction with a visual inspection by the Unit Fire Protection Engineer, has determined the door has retained its fire resistance rating. Therefore the door was operable.

There were no operator actions required as a result of this event. There were no automatic or manually initiated safety systems activated during this event.

II. Cause of Event

The root cause of the event is administrative personnel error. The Millstone Unit Two Barrier Boundary drawings correctly identify the barrier as a technical specification required fire barrier however, both the surveillance procedure and a station form had inadvertently omitted the door from the proper classification.

III. Analysis of Event

This event is being reported pursuant to 10CFR50.73(a)(2)(i)(B), any operation or condition prohibited by the plant's Technical Specifications.

The fire door was determined to be operable. Thus, there was no safety significance as a result of this event.

IV. Corrective Action

As soon as it was verified that the subject door as required by Technical Specification the Shift Supervisor was notified. Subsequently, the door was added to the appropriate surveillance procedure and successfully inspected.

Additionally, an investigation into the method of administrative control for all door attributes was completed. No additional reportable issues were identified.

V. Additional Information

None

No Similar LERs.

EIIS Codes

NF-DR-C176