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SERIAL: BSEP 95-0316
10 CFR 50.90
TSC 95TSB34

SEP 13 1995

U. S. Nuclear Regulatory Commission
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Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324/LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

Gentlemen:

In accordance with the Code of Federal Regulations, Title 10, Parts 50.90 and 2.101, Carolina Power & Light Company hereby requests a revision to the Technical Specifications for the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2. The proposed license amendments included administrative changes to revise Technical Specifications 3/4.6.1.2 and 3/4.6.1.3 by:

- Removing detailed requirements describing Type B and C testing for primary containment isolation valves and penetrations and instead adopting wording which references 10 CFR Part 50, Appendix J, and
- Removing detailed requirements describing leakage testing for primary containment air locks and instead adopting wording which references 10 CFR Part 50, Appendix J.

Enclosure 1 provides a detailed description of the proposed changes and the basis for the changes.

Enclosure 2 details the basis for the Company's determination that the proposed changes do not involve a significant hazards consideration.

Enclosure 3 provides an environmental evaluation which demonstrates that the proposed amendments meet the eligibility for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental assessment needs to be prepared in connection with the issuance of the amendment.

Enclosure 4 provides page change instructions for incorporating the proposed revisions.

Enclosure 5 provides the marked-up Technical Specification pages for Unit 1.

Enclosure 6 provides the marked-up Technical Specification pages for Unit 2.

Enclosure 7 provides the typed Technical Specification pages for Unit 1.

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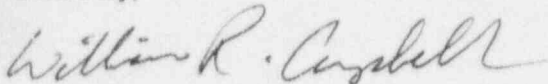
Enclosure 8 provides the typed Technical Specification pages for Unit 2.

Carolina Power & Light Company is providing, in accordance with 10 CFR 50.91(b), Mr. Dayne H. Brown of the State of North Carolina with a copy of the proposed license amendments.

In order to allow time for procedure revision, orderly incorporation into copies of the Technical Specifications, CP&L requests that the proposed amendment, once approved by the NRC, be issued with an effective date to be no later than 60 days from the issuance date of the license amendments.

Please refer any questions regarding this submittal to Mr. George Honma at (910) 457-2741.

Sincerely,



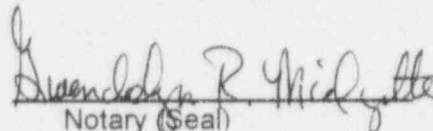
William R. Campbell

WRM/wrm

Enclosures:

1. Basis for Change Request
2. 10CFR50.92 Evaluation
3. Environmental Considerations
4. Page Change Instructions
5. Marked-up Technical Specification Pages - Unit 1
6. Marked-up Technical Specification Pages - Unit 2
7. Typed Technical Specification Pages - Unit 1
8. Typed Technical Specification Pages - Unit 2

William R. Campbell, having been first duly sworn, did depose and say that the information contained herein is true and correct to the best of his information, knowledge and belief; and the sources of his information are officers, employees, and agents of Carolina Power & Light Company.



Notary (Seal)

My commission expires *Dec 12, 1996*

pc: Mr. D. H. Brown, State of North Carolina
Mr. S. D. Ebner, Regional Administrator, Region II
Mr. D. C. Trimble, NRR Project Manager - Brunswick Units 1 and 2
Mr. C. A. Patterson, Brunswick NRC Senior Resident Inspector
The Honorable H. Wells, Chairman - North Carolina Utilities Commission

ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

BASIS FOR CHANGES

PROPOSED CHANGE 1:

Current Requirement

Technical Specification 4.6.1.2 states:

The primary containment leakage rates shall be demonstrated in accordance with the schedule and criteria specified in 10 CFR 50, Appendix J, as modified by approved exemptions. The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in 10 CFR 50, Appendix J.

Technical Specification 4.6.1.2.d states:

Type B and C tests shall be conducted with gas at P_a , 49 psig, at intervals no greater than 24 months except for tests involving:

1. Air locks,
2. Main steam line isolation valves,

Technical Specification 4.6.1.2.h states:

The provisions of Specification 4.0.2 are not applicable to 24 month surveillance intervals.

Proposed Change

The proposed change would revise Technical Specification 4.6.1.2 by removing detailed requirements describing Type B and C testing and instead adopting wording which references 10 CFR Part 50, Appendix J. The adoption of generalized wording that references 10 CFR Part 50, Appendix J instead of restating the Appendix J requirements is consistent with the approach used in the BWR-4 Improved Technical Specifications (NUREG-1433) relative to primary containment leakage rate testing.

The specific revisions to Technical Specification 4.6.1.2 to implement the proposed changes are as follows:

1. Replace Technical Specification 4.6.1.2.d with the following:

Type B and C tests shall be conducted in accordance with 10 CFR 50, Appendix J, as modified by approved exemptions, except for tests involving main steam line isolation valves.

2. Delete Technical Specification 4.6.1.2.h.

Basis For Proposed Change

The proposed license amendments revise Brunswick Plant Technical Specification 4.6.1.2 by removing the schedule requirements for Type B and C leakage rate tests to be performed at intervals no greater than 24 months. In addition, the proposed license amendments remove the test pressure criteria for these Type B and C local leakage rate tests. Instead, the Technical Specifications should be revised to reference the containment leakage testing requirements of Appendix J to 10 CFR Part 50.

Appendix J to 10 CFR Part 50 requires the following:

- Type B tests, except for tests for air locks, shall be performed during reactor shutdown for refueling, or other convenient intervals, but in no case at intervals greater than 2 years. (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 2.(a), "Type B tests").
- Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years. (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 3, "Type C tests").

As stated in the "Current Requirement" section above, Technical Specification 4.6.1.2.d requires that Type B and C tests be conducted at intervals no greater than 24 months except for tests involving air locks and main steam line isolation valves. The proposed license amendments will maintain in the Technical Specifications the same scheduling requirements for Type B and C leakage rate tests as that currently required by the Technical Specifications and 10 CFR Part 50, Appendix J, but will remove the detailed discussion of Type B and C testing schedules and testing methods.

The proposed license amendments will remove the test pressure criteria for these Type B and C local leakage rate tests. Appendix J to 10 CFR Part 50 requires the following:

- All pre-operational and periodic Type B tests shall be performed by local pneumatic pressurization of the containment penetrations, either individually or in groups, at a pressure not less than P_a . (Section III, "Leakage Testing Requirements"; Paragraph B, "Type B tests"; Part 2, "Test pressure").
- Valves, unless pressurized with fluid (e.g., water, nitrogen) from a seal system, shall be pressurized with air or nitrogen at a pressure of P_a . (Section III, "Leakage Testing Requirements"; Paragraph C, "Type C tests"; Part 2(a), "Test pressure").

As stated above, Technical Specification 4.6.1.2.d currently requires that Type B and C tests be conducted with gas at P_a , 49 psig, except for tests involving air locks and main steam line isolation valves. Thus, the proposed license amendments will maintain in the Technical Specifications the same test pressure requirements for Type B and C local leakage rate tests as that currently required by the Technical Specifications and by 10 CFR Part 50, Appendix J.

In addition, the Technical Specification 4.6.1.2.d.1 reference to a test exception for air locks is also being deleted. Specific requirements for the test frequency and test methods applicable to containment air locks are contained in Technical Specification 3/4.6.1.3 and Appendix J of 10 CFR Part 50.

A comparison of the existing Brunswick Plant Technical Specification requirements and the requirements of 10 CFR Part 50, Appendix J is provided in Table 1. As shown in this comparison, the test intervals and methods for Type B and C leakage rate tests specified in the current Brunswick Plant Technical Specifications are identical to corresponding requirements in 10 CFR 50, Appendix J.

The proposed license amendments will also remove Technical Specification 4.6.1.2.h, which states that the 25 percent surveillance interval extension allowed by Technical Specification 4.0.2 is not applicable to 24 month surveillance intervals. Since Technical Specification 4.6.1.2.d will no longer specify the surveillance interval for Type B and C leakage rate tests, the Technical Specification 4.0.2 prohibition for this test interval is no longer needed and should be deleted from Technical Specification 4.6.1.2.h. Appendix J to 10 CFR Part 50 currently specifies that Type B and C tests be conducted at intervals no greater than 2 years (Section III, Paragraph D.2(a) for Type B tests and Section III, Paragraph D.3 for Type C tests). Since the proposed license amendments will incorporate references to Appendix J of 10 CFR Part 50 rather than repeating the actual provisions of 10 CFR Part 50, Appendix J, and since Appendix J of 10 CFR Part 50 does not allow for extensions of specified test intervals other than through the exemption process, the Technical Specification 4.6.1.2.h statement prohibiting the use of 24 month surveillance interval extensions (i.e., Type B and C tests referenced in Technical Specification 4.6.1.2.d) is not necessary and is being deleted.

PROPOSED CHANGE 2:

Current Requirement

Technical Specification 4.6.1.3.a states:

Each primary containment air lock shall be demonstrated OPERABLE:

- a. By verifying the seal leakage rate to be less than or equal to 5 scf per hour when the gap between the door seals is pressurized to 10 psig:
 1. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours...

Technical Specification 4.6.1.3.b states:

Each primary containment air lock shall be demonstrated OPERABLE:

- b. By conducting an overall air lock leakage test at P_a , 49 psig, and by verifying that the overall air lock leakage is within its limits:
 1. At least once per six months[#], and

2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when maintenance (except for seal replacement) has been performed on the air lock that would affect the air lock sealing capability.

Proposed Change

The proposed change would revise Technical Specification 4.6.1.3 by removing detailed requirements describing containment air lock testing and instead adopting wording which references 10 CFR Part 50, Appendix J. The specific revisions to Technical Specification 4.6.1.3 to implement the proposed changes are as follows:

1. Modify Technical Specification 4.6.1.3.a as follows:
 - a. By verifying the seal leakage rate to be less than or equal to 5 scf per hour when the gap between the door seals is pressurized to 10 psig:
 1. As specified in Appendix J of 10 CFR Part 50, as modified by approved exemptions.

Technical Specifications 4.6.1.3.a.2 and 4.6.1.3.a.3 would be left unchanged.

2. Modify Technical Specification 4.6.1.3.b as follows:
 - b. By conducting an overall air lock leakage test in accordance with 10 CFR 50, Appendix J, as modified by approved exemptions, and prior to establishing PRIMARY CONTAINMENT INTEGRITY when maintenance (except for seal replacement) has been performed on the air lock that could affect the air lock sealing capability. The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in 10 CFR 50, Appendix J.
3. Relocate to Technical Specification 4.6.1.3.b the footnote prohibiting the application of Technical Specification 4.0.2 to test intervals specified in 10 CFR 50, Appendix J.

Appendix J of 10 CFR Part 50 requires the following:

- Air locks shall be tested prior to initial fuel loading and at 6-month intervals thereafter at an internal pressure not less than P_a (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 2, "Type B tests," Subpart (b)(i))
- Air locks opened during periods when containment integrity is not required by the plant's Technical specifications shall be tested at the end of such periods at not less than P_a . (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 2, "Type B tests," Subpart (b)(ii))
- Air locks opened during periods when containment integrity is required by the plant's Technical Specifications shall be tested within 3 days after being opened. For air lock doors opened more frequently than at least once every 3 days, the air lock shall be tested at least once every 3 days during the period of frequent openings. For air lock doors having testable seals, testing the seals fulfills the 3-day test requirements. In the event that the testing for this 3-day interval cannot be at P_a , the test pressure shall be as stated

in the Technical Specifications. Air lock door seal testing shall not be substituted for the 6-month test of the entire air lock at not less than P_a . (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 2, "Type B tests," Subpart (b)(iii))

- The acceptance criteria for air lock testing shall be stated in the Technical Specifications. (Section III, "Leakage Testing Requirements"; Paragraph D, "Periodic retest schedule"; Part 2, "Type B tests," Subpart (b)(iv))

Technical Specification 4.6.1.3.a.1 requires that the primary containment air lock door seals be tested within 72 hours after each air lock door closing, except when the air lock is being used for multiple entries. In the case of multiple air lock entries, Technical Specification 4.6.1.3.a.1 also requires the primary containment air lock door seals be tested at least every 72 hours. Referencing Appendix J of 10 CFR Part 50 in Technical Specification 4.6.1.3.a.1 instead of including these detailed Appendix J requirements in the Technical Specifications will maintain the same primary containment air lock testing frequency requirements as those stipulated in Appendix J of 10 CFR Part 50.

As stated in the "Current Requirement" section above, Technical Specification 4.6.1.3.b requires that an overall air lock leakage test be conducted at intervals no greater than 6 months and prior to establishing primary containment integrity following maintenance activities affecting the air lock seals. Technical Specification 4.6.1.3.b also states that the test shall be performed at a test pressure of P_a (49 psig). Referencing Appendix J of 10 CFR Part 50 in Technical Specification 4.6.1.3.b instead of including these detailed Appendix J requirements in the Technical Specifications will maintain the same primary containment air lock testing frequency and test pressure requirements as those stipulated in Appendix J of 10 CFR Part 50.

Technical Specification 3.6.1.3.b states that the acceptance criteria for primary containment air lock leakage rate is less than or equal to $0.05 L_a$ at P_a , 49 psig. Appendix J of 10 CFR Part 50 requires that the acceptance criteria for air lock testing be stated in the Technical Specifications. Therefore, the acceptance criteria for air lock testing is being retained in Technical Specification 3.6.1.3.b.

A comparison of the existing Brunswick Plant Technical Specification requirements for containment air lock testing and the requirements of 10 CFR Part 50, Appendix J is provided in Table 2. This comparison demonstrates that the test intervals and test methods for containment air lock testing specified in the existing Brunswick Plant Technical Specifications are identical to the corresponding requirements in 10 CFR 50, Appendix J.

TABLE 1

COMPARISON OF EXISTING TECHNICAL SPECIFICATION
REQUIREMENTS PERTAINING TO TYPE B AND C TESTING
PER 10 CFR 50, APPENDIX J

EXISTING TECHNICAL SPECIFICATION	10 CFR 50, APPENDIX J
<p>Technical Specification 4.6.1.2.d specifies the testing frequency for performing Type B and C tests shall be no greater than 24 months except for tests involving air locks and the main steam line isolation valves.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph D, "Periodic retest schedule"; Part 2.(a), "Type B tests" specifies identical testing frequencies for performing Type B containment local leakage rate tests.</p> <p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph D, "Periodic retest schedule"; Part 3, "Type C tests" specifies identical testing frequencies for performing Type C containment local leakage rate tests.</p>
<p>Technical Specification 4.6.1.2.d specifies the testing pressure to be used in conducting periodic Type B and C tests.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph B, "Type B tests"; Part 2, "Test pressure" specifies identical testing pressure requirements for the periodic Type B tests.</p> <p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph C, "Type C tests"; Part 2(a), "Test pressure" specifies identical testing pressure requirements for the periodic Type C tests.</p>
<p>Technical Specification 4.6.1.2.h specifies that the 25 percent extension of the surveillance interval permitted by Technical Specification 4.0.2 is not applicable to the 24 month surveillance frequencies required by 10 CFR 50, Appendix J.</p>	<p>10 CFR 50, Appendix J requires that the 24 month surveillance tests being performed "at intervals no greater than 2 years" and does not contain provisions for extending the specified surveillance frequencies.</p>

TABLE 2

COMPARISON OF EXISTING TECHNICAL SPECIFICATION
REQUIREMENTS PERTAINING TO AIR LOCK TESTING
PER 10 CFR 50, APPENDIX J

EXISTING TECHNICAL SPECIFICATION	10 CFR 50, APPENDIX J
<p>Technical Specification 3.6.1.3.b specifies that acceptance criteria for an overall air lock leakage rate test as being "less than or equal to 0.05 L_a at P_a, 49 psig.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph B, "Type B tests"; Part 2(iv), "Type B tests" specifies that the acceptance criteria for air lock testing be stated in the plant Technical Specifications.</p>
<p>Technical Specification 4.6.1.3.a.1 specifies that primary containment air lock seal leakage rate testing shall performed within 72 hours following each closing of the air lock, except when the air lock is being used for multiple entries and then at least once per 72 hours.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph Paragraph D, "Periodic retest schedule"; Part 2.(iii), "Type B tests" specifies identical testing frequencies (i.e., 3 days) for performing containment air lock seal leakage rate tests.</p>
<p>Technical Specification 4.6.1.3.b specifies the testing pressure to be used in conducting an overall containment air lock leakage test.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph B, "Type B tests"; Part 2, "Test pressure" and Section III (Leakage Testing Requirements), Paragraph D, "Periodic retest schedule"; Part 2, "Type B tests," Subpart (b)(i) specify identical air lock leakage testing pressure requirements.</p>
<p>Technical Specification 4.6.1.3.b.1 specifies that primary containment air lock testing performed at least once per six months.</p>	<p>10 CFR 50, Appendix J, Section III (Leakage Testing Requirements), Paragraph B, "Type B tests"; Part 2(b)(i), "Type B tests" specifies identical air lock leakage testing frequency requirements.</p>
<p>Footnote "#" for Technical Specification 4.6.1.3.b specifies that the 25 percent extension of the surveillance interval permitted by Technical Specification 4.0.2 is not applicable to the surveillance frequencies required by 10 CFR 50, Appendix J.</p>	<p>10 CFR 50, Appendix J does not include provisions for extending the 6-month surveillance test interval provided therein.</p>

ENCLOSURE 2

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

10 CFR 50.92 EVALUATION

The Commission has provided standards in 10 CFR 50.92 for determining whether a significant hazards consideration exists. A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety. Carolina Power & Light Company has reviewed these proposed license amendments and believes that their adoption would not involve a significant hazards consideration. The basis for this determination follows.

1. The proposed license amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed license amendments remove the detailed technical and schedular information pertaining to primary containment local leakage rate testing (e.g., Type B and Type C testing) and testing of the primary containment air lock and associated door seals from the Technical Specifications. Instead, these proposed license amendments will reference the corresponding requirements that are located in the Appendix J of 10 CFR Part 50. As such, the proposed amendments are an administrative change since the actual requirements for the performance of primary containment local leakage rate testing and primary containment air lock testing are not being changed. No safety-related equipment, safety function, or plant operations will be altered as a result of the proposed amendments. The change does not affect the design, materials, or construction standards of the primary containment and primary containment air locks nor the test methods, test acceptance criteria, or testing frequencies applicable to primary containment local leakage rate testing and primary containment air lock leakage rate testing. Based on the above, the proposed license amendments do not create a significant increase in the probability or consequences of an accident previously evaluated.
2. The proposed license amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated. As stated above, no safety-related equipment, safety function, or plant operations will be altered as a result of the proposed change. The proposed amendments do not change the primary containment design or the test methods, test acceptance criteria, or testing frequencies for primary containment local leakage rate testing and primary containment air lock leakage rate testing. As such, the proposed license amendments cannot create the possibility of a new or different kind of accident from any accident previously evaluated.
3. The proposed license amendments do not involve a significant reduction in a margin of safety. The proposed amendments do not involve any changes to the test methods, acceptance criteria, or testing frequency for primary containment local leakage rate testing and primary containment air lock leakage rate testing. Thus, the proposed amendments will not affect the ability of the primary containment and primary containment air lock to perform their intended safety function and no margins of safety, as defined by the plant's

accident analyses, are impacted. Primary containment local leakage rate testing and primary containment air lock leakage rate testing will continue to be performed in accordance with the regulatory requirements of Appendix J to 10 CFR Part 50. Based on the above reasoning, the proposed license amendments do not involve a significant reduction in the margin of safety.

ENCLOSURE 3

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

ENVIRONMENTAL CONSIDERATIONS

10 CFR 51.22(c)(9) provides criterion for and identification of licensing and regulatory actions eligible for categorical exclusion from performing an environmental assessment. A proposed amendment to an operating license for a facility requires no environmental assessment if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant hazards consideration, (2) result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite, or (3) result in an increase in individual or cumulative occupational radiation exposure. Carolina Power & Light Company has reviewed this request and believes that the proposed license amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(c), no environmental impact statement or environmental assessment needs to be prepared in connection with the issuance of the proposed license amendments. The basis for this determination follows.

1. These proposed license amendments do not involve a significant hazards consideration, as shown in Enclosure 2.
2. The proposed license amendments do not result in a significant change in the types or a significant increase in the amounts of any effluent that may be released offsite. The proposed license amendments do not introduce any new equipment nor does it require any existing equipment or systems to perform a different type of function than they are presently designed to perform. The proposed license amendments do not alter the testing methods, testing acceptance criteria, or testing frequencies associated with primary containment local leakage rate testing. Therefore, the primary containment local leakage rate testing being performed will continue to ensure that the consequences of any previously evaluated accident do not increase. Therefore, CP&L has concluded that there will not be a significant increase in the types or amounts of any effluent that may be released offsite and, as such, does not involve irreversible environmental consequences beyond those already associated with normal operation.
3. The proposed license amendments do not change the frequency, duration, methods, or acceptance criteria for performance of primary containment local leakage rate testing. Since the containment local leakage rate testing will be unchanged, the amendments do not result in an increase in individual or cumulative occupational radiation exposure.

ENCLOSURE 4

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

<u>PAGE CHANGE INSTRUCTIONS</u>	
<u>UNIT 1</u>	
Removed page	Inserted page
3/4 6-3	3/4 6-3
3/4 6-5	3/4 6-5
B 3/4 6-1	B 3/4 6-1
B 3/4 6-2	B 3/4 6-2

<u>PAGE CHANGE INSTRUCTIONS</u>	
<u>UNIT 2</u>	
Removed page	Inserted page
3/4 6-3	3/4 6-3
3/4 6-5	3/4 6-5
B 3/4 6-1	B 3/4 6-1
B 3/4 6-2	B 3/4 6-2

ENCLOSURE 5

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT LEAKAGE RATE TESTING

MARKED-UP TECHNICAL SPECIFICATION PAGES - UNIT 1