

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

September 15, 1995
LIC-95-0172

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555


Reference: Docket No. 50-285

SUBJECT: August 1995 Monthly Operating Report (MOR)

Enclosed please find the August 1995 MOR for Fort Calhoun Station (FCS)
Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



T. L. Patterson
Division Manager
Nuclear Operations

TLP/d11

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

AUGUST 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of August, Fort Calhoun Station (FCS) operated at a nominal 100% power until 1114 hours on August 24th. At that time, the plant experienced an automatic reactor trip and associated diesel generator (DG) start. The trip occurred during the performance of the Diverse Scram System Actuation Relay Operability Test, when a licensed operator inadvertently repositioned the incorrect switch from TEST to NORMAL. All systems responded normally to the trip with the exception of the DG-1 accelerating to full speed vice idlespeed. After evaluation of the DG-1 electrical system conditions, this DG-1 full speed event was reported on August 28th in accordance with 10 CFR 50.72(b)(1)(ii)(B). The reactor was taken critical at 0220 hours on August 26th and was generating at 100% capacity by August 29th. Reactor criticality was delayed because of a failure of the motor brake on one of the Control Rod Drive Motors.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of August, no Power Operated Relief Valves (PORV) or Primary System safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The Reactor Coolant System (RCS) leak rate was steady until August 26th when the amount of "Known" leakage increased above its nominal value for the month. This increase corresponded with an increase in Control Element Drive Mechanism (CEDM) #15 temperature. The increase in CEDM #15 temperature was noted when the rod was exercised while performing Technical Specification required surveillance testing following the August 24th reactor trip.

The increased RCS leakage, as seen in the "Known" leak rate, is collected by the Reactor Coolant Drain Tank (RCDT). The RCDT receives leakage from a large number of sources in containment including the CEDM leak-offs. From inspection activities, the increase in leakage appears to be completely contained in the RCDT with no increase noted in the containment sump or visible signs of leakage in containment. The "Known" leak rate stabilized at a leak rate of approximately 0.32 gpm with a "Total" leak rate of slightly above 0.40 gpm during this report period.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF AUGUST 1995

- Installed a temporary modification on Reactor Coolant Temperature Indicator D/T1-122H due to a faulty RTD
- Replaced Raw Water Pump AC-10A with a rebuilt unit
- Replaced the motor brake package on CEDM RC-10-08
- Installed Firmware Upgrade to QSPDS Channels A & B

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
 UNIT FORT CALHOUN STATION
 DATE SEPTEMBER 06, 1995
 COMPLETED BY D. L. LIPPY
 TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: AUGUST 1995 NOTES
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A
9. Power Level to which restricted, if any (Net MWe): N/A
10. Reasons for restrictions, if any:
N/A

	THIS MONTH -----	YR-TO-DATE -----	CUMULATIVE -----
11. Hours in Reporting Period.....	744.0	5831.0	192265.0
12. Number of Hours Reactor was Critical	705.0	4361.1	150779.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	691.6	4277.2	149051.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	989936.2	6154493.1	198303193.6
17. Gross Elec. Energy Generated (MWH)..	320816.0	2045918.0	65450800.2
18. Net Elec. Energy Generated (MWH)....	304623.0	1948616.0	62440408.3
19. Unit Service Factor.....	93.0	73.4	77.5
20. Unit Availability Factor.....	93.0	73.4	77.5
21. Unit Capacity Factor (using MDC Net)	85.7	69.9	70.2
22. Unit Capacity Factor (using DER Net)	85.7	69.9	68.6
23. Unit Forced Outage Rate.....	5.4	5.8	4.1

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
N/A
 25. If shut down at end of report period, estimated date of startup: _____
 26. Units in test status (prior to comm. oper.): Forecast Achieved
- | | | | | | |
|--|-----|--|--|--|--|
| INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION | N/A | | | | |
|--|-----|--|--|--|--|

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	SEPTEMBER 06, 1995
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH AUGUST 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	466	17	466
2	468	18	464
3	468	19	463
4	466	20	465
5	465	21	466
6	465	22	466
7	467	23	466
8	465	24	209
9	463	25	0
10	463	26	13
11	463	27	98
12	462	28	297
13	461	29	461
14	461	30	463
15	463	31	465
16	466		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun St.
 DATE September 7, 1995
 COMPLETED BY D. L. Lippy
 TELEPHONE (402) 533-6843

REPORT MONTH August 1995

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
95-04	950824	F	39.1	H	3	95-005	ZZ	ZZZZZZ	At 1114 hours on August 24, 1995 the plant experienced a reactor trip and associated diesel generator (DG) start. The trip occurred during the performance of the Diverse Scram System Actuation Relay Operability Test, when a licensed operator inadvertently repositioned the incorrect switch from TEST to NORMAL. All systems responded normally to the trip with the exception of DG-1 going to full speed vice idle speed. Corrective action includes institutionalizing the use of peer checking for switch manipulations that could immediately cause a reactor trip. The reactor was taken critical at 0220 hours on August 26th.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending August 31, 1995

1. Scheduled date for next refueling shutdown. September 21, 1996
2. Scheduled date for restart following refueling. November 2, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? No
 - a. If answer is yes, what, in general, will these be? N/A
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. No
 - c. If no such review has taken place, when is it scheduled? Prior to November 1996
4. Scheduled date(s) for submitting proposed licensing action and support information. No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. N/A
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 618 Assemblies
 - c) spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 2007 Outage

Prepared by John A. Williams

Date 9-8-95