



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

September 15, 1995
BECO Ltr. #95-097

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

AUGUST 1995 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


L.J. Olivier

WJM/laa/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE 9/15/95
 COMPLETED BY: W.J. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

- | | | |
|----|---|-------------|
| 1. | Unit Name | Pilgrim I |
| 2. | Reporting Period | August 1995 |
| 3. | Licensed Thermal Power (MWt) | <u>1998</u> |
| 4. | Nameplate Rating (Gross MWe) | <u>678</u> |
| 5. | Design Electrical Rating (Net MWe) | <u>655</u> |
| 6. | Maximum Dependable Capacity (Gross MWe) | <u>696</u> |
| 7. | Maximum Dependable Capacity (Net MWe) | <u>670</u> |
| 8. | If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons: | |

NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
 10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744.0</u>	<u>5831.0</u>	<u>199223.0</u>
12. Hours Reactor Critical	<u>744.0</u>	<u>4137.0</u>	<u>123337.1</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>4033.8</u>	<u>118894.9</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1480401.0</u>	<u>7799783.0</u>	<u>210554999.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>505570.0</u>	<u>2678140.0</u>	<u>71297814.0</u>
18. Net Electrical Energy Generated(MWH)	<u>486906.0</u>	<u>2576385.0</u>	<u>68534162.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>69.2</u>	<u>59.7</u>
20. Unit Availability Factor	<u>100.0</u>	<u>69.2</u>	<u>59.7</u>
21. Unit Capacity Factor (Using MDC Net)	<u>97.7</u>	<u>65.9</u>	<u>51.3</u>
22. Unit Capacity Factor (Using DER Net)	<u>99.9</u>	<u>67.5</u>	<u>52.5</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>6.1</u>	<u>12.2</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each)	NONE		
25. If Shutdown at End of Report Period, Estimated Date of Startup - Unit Operating			

AVERAGE DAILY UNIT POWER LEVEL

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MONTH August 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	662	17	658
2	661	18	652
3	660	19	648
4	659	20	576
5	658	21	653
6	657	22	654
7	655	23	652
8	654	24	655
9	656	25	656
10	656	26	656
11	659	27	658
12	661	28	658
13	660	29	659
14	659	30	658
15	658	31	658
16	660		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR AUGUST 1995

The unit started the reporting period at approximately 100 percent Core Thermal Power (CTP) where it was essentially maintained for the remainder of the reporting period. A brief power reduction was initiated on August 19, 1995 to facilitate a rod pattern adjustment.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF AUGUST 1995

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo. dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: March 29, 1997.
3. Scheduled date for restart following next refueling: May 12, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1765 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

DOCKET NO: 50-293
 NAME: Pilgrim I
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 COMPLETED BY: W.J. Munro
 TELEPHONE: (508) 830-8474
 REPORT MONTH: August 1995

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Primary Containment System/Reactor Water Cleanup System	MO-1201-80, Reactor Water Cleanup System Pump Suction Inboard Isolation Valve	Due to a wiring error discovered by engineering personnel the "as found" condition would not allow the valve to automatically travel to the fully closed position in the event of an isolation signal.	Contractor electrician personnel error during a modification installation performed in 1993. (Refer to associated LER)	The wiring was restored to the design configuration. Post work testing was completed satisfactorily.	Training for maintenance personnel on the specific event and an engineering review of past wiring discrepancies to determine common causes; and field verifications of potentially impacted components identified as a result of the review.	95-008-00

Core Spray System	MO-1400-4A Test Return Valve	MO-1400-4A was found with a lifted lead that should have been terminated. The lifted lead would have resulted in the valve stopping in the close direction on the limit switch instead of the torque switch. MO-1400-4A would have closed to approximately 96% to 98% of its closed position. This however, would have satisfied the design function.	Electrician personnel error. Also a contributing factor was the clarity of the design drawing for MO-1400-4A.	The lifted lead was landed correctly. Testing was satisfactorily completed.	Field walkdowns and review of other Core Spray design drawings were conducted. (Refer to associated LER)	95-008-00
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UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE

There were no unit shutdowns or significant power reductions during the reporting period.

- | | | | |
|---------------------|--|--|--|
| 1 | 2 | 3 | 4&5 |
| F-Forced
S-Sched | A-Equip Failure
B-Main or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training
& License Examination
F-Admin
G-Operator Error
H-Other | 1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other | Exhibit F & H
Instructions for
Preparations of
Data Entry Sheet
Licensee Event Report
(LER) File (NUREG-1022) |