NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

UPDATE REPORT
PREVIOUS REPORT DATE 01/06/83

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
0 1	LICENSEE CODE (2)	G A E I H 2
	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 0 1	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	0 5 0 0 0 3 6 6
	EVENT DATE (8)	1 2 0 8 8 2
	REPORT DATE (9)	0 6 2 0 8 4

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During performance of a routine panel surveillance, it was discovered
0 3 that "A" loop RHR discharge pressure was zero; investigation showed
0 4 that the loop was not filled with water as required by Tech. Specs.
0 5 3/4.5.3.2. The redundant "B" loop of RHR remained operable during this
0 6 event. The health and safety of the public were not affected by this
0 7 non-repetitive event.
0 8

SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
C F	X	Z	Z Z Z Z Z Z	X	Z

LER/RO REPORT NUMBER (17)	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	--	1 4 0	/	0 3	X	—	1

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
X	Z	Z	Z	0 0 0 0	Y	N	A	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event is unknown. Initially, a check valve was
1 1 suspected of being the cause of this event. However, an inspection of
1 2 the check valve revealed no problem. The RHR line was filled and vented
1 3 in approximately 1 hour after the event was detected. Consequently,
1 4 RHR's "A" loop was returned to service on 12/08/82.

Line	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
1 5	C	0 1 2	NA	A	Routine Panel Surveillance

Line	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z	Z	NA	NA

PERSONNEL EXPOSURES

Line	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
1 7	0 0 0	Z	NA

PERSONNEL INJURIES

Line	NUMBER (40)	DESCRIPTION (41)
1 8	0 0 0	NA

LOSS OF OR DAMAGE TO FACILITY (43)

Line	TYPE (42)	DESCRIPTION
1 9	Z	NA

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PUBLICITY

Line	ISSUED (44)	DESCRIPTION (45)
2 0	N	NA

8406250077 840620
PDR ADOCK 05000366
S PDR

NRC USE ONLY

NAME OF PREPARER S. B. Tipps PHONE: (912)367-7851

NARRATIVE REPORT
FOR LER 50-366/1982-140, Rev. 1
UPDATE REPORT - PREVIOIS REPORT DATE 01/06/83

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b because the event showed that the unit was not meeting the requirements of Tech. Specs. section 3.5.3.2.a.

Plant conditions at the time of the event(s):

At the time of this event on 12/08/82 the unit was in startup and hot standby and reactor power was at 300 MWt (approximately 12% power).

Detailed description of the event(s):

While performing the "SCRAM RECOVERY" procedure (HNP-2-1010), surveillance personnel noticed that the Residual Heat Removal (RHR) "A" loop discharge pressure was 0 PSIG. Further investigation revealed that the discharge piping wasn't filled. This event is contrary to the requirements of Tech. Specs. section 3.5.3.2.a because the system piping wasn't filled as required by Technical Specifications section 4.5.3.2.a.1.

Consequences of the event(s):

The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

The "B" loop of RHR remained operable during this event.

Justification for continued operation:

The "A" RHR loop was filled, vented, and returned to operable status within approximately 1 hour after the event was detected.

If repetitive, number of previous LER:

This is a non-repetitive event.

Impact to other systems and/or Unit:

This event had no effect on any other Unit 2 system. This event did not affect Unit 1.

Cause(s) of the event(s):

The cause of this event is unknown. Initially, the "A" RHR pump's discharge line check valve 2E11-F031A was suspected of being the cause of the event due to a possible leak. However, on 04/28/83, an inspection was performed on the check valve and no problem was detected.

Immediate Corrective Action:

2E11-F031A was checked to assure a proper seat. The RHR line was then filled and vented. The "A" loop of RHR was returned to service on 12/08/82, approximately 1 hour after the event was detected.

Supplemental Corrective Action:

On 04/28/83, an inspection was performed on the suspected leaking check valve. However, no evidence of leaking or malfunction was detected.

Scheduled (future) corrective action:

N/A

Action to prevent recurrence (if different from corrective actions):

N/A

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444

Georgia Power

Edwin I. Hatch Nuclear Plant

June 20, 1984
GM-84-204

PLANT E. I. HATCH
Licensee Event Report
Docket No. 5C-366

United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1982-140, Rev. 1. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.9.b.

H. C. Nix
General Manager

HCN/SBT/djs

xc: R. J. Kelly
R. E. Conway
J. T. Beckham, Jr.
P. D. Rice
K. M. Gillespie
S. B. Tipps
R. D. Baker
Control Room
Document Control

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