

North Atlantic Energy Service Corporation P.O. Box 300 Seabrook, NH 03874 (603) 474-9521, Fax (603) 474-2987

The Northeast Utilities System

Ted C. Feigenbaum Senior Vice President & Chief Nuclear Officer

NYN-95072

September 13, 1995

United States Nuclear Regulatory Commission Washington, D.C. 20555

Attention:

Document Control Desk

Keference:

Facility Operating License No. NPF-86, Docket No. 50-443

Subject:

Licensee Event Report (LER) No. 95-005-00: "Non-Compliance with Technical

Specification 3.3.1 Action Requirements"

Gentlemen:

Enclosed please find Licensee Event Report (LER) No. 95-005-00 for Seabrook Station. This submittal documents an event which occurred on July 31, 1995. This event is being reported pursuant to 10CFR50.73(a)(2)(i). A supplemental Licensee Event Report is expected to be submitted by October 15, 1995.

Should you require further information regarding this matter, please contact Mr. James M. Peschel, Regulatory Compliance Manager, at (603) 474-9521, extension 3772.

Very truly yours,

Feigenbaum

TCF:EWM/act

Enclosures: NRC Forms 366/366A

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cc: Mr. Thomas T. Martin
Regional Administrator
United States Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Mr. Albert W. De Agazio, Sr. Project Manager Project Directorate I-4 Division of Reactor Projects U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Mr. John Macdonald NRC Senior Resident inspector P.O. Box 1149 Seabrook, NH 03874

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APPROVED BY OMB NO. 31 EXPIRES 5/31/95 NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (5-92)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. LICENSEE EVENT REPORT (LEF.) FORWARD COMMENTS REGARDING BURDEN ESTIMATE THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNEB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF (See reverse for required number of digits/characters for each block) MANAGEMENT AND BUDGET, DC 20503 WASHINGTON . FACILITY NAME (1) DOCKET NUMBER (2) PAGE (3) 05000443 1 OF 4 Seabrook Station TITLE (4) Non-Compliance with Technical Specification 3.3.1 Action Requirements OTHER FACILITIES INVOLVED (8) EVENT DATE (5) LER NUMBER (6) REPORT DATE (7) FACILITY NAME DOCKET NUMBER SEQUENTIAL REVISION YEAR DAY YEAR MONTH DAY MONTH YEAR 05000 NUMBER NUMBER FACILITY NAME DOCKET NUMBER 95 13 07 31 95 05 00 09 05000 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR \$: (Check one or more) **OPERATING** 1 73.71(b) MODE (9) 20.405(c) 50.73(a)(2)(iv) 20.402(b) 50.36(c)(1) 50.73(a)(2)(v) 73.71(c) 20.405(a)(1)(i) POWER 100 50.73(a)(2)(vii) LEVEL (10) 20.405(a)(1)(ii) 50.36(c)(2) (Specify in 50.73(a)(2)(viii)(A) 50.73(a)(2)(i) 20.405(a)(1)(iii) Abstract below 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B) 20.405(a)(1)(iv) and in Text, 50.73(a)(2)(x) 50.73(a)(2)(iii) 20.405(a)(1)(v) NRC Form 366A) LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER (Include Area Code) (603) 474-9521, James M. Peschel, Regulatory Compliance Manager 3772 COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) REPORTABLE REPORTABLE MANUFACTURER COMPONENT COMPONENT MANUFACTURER CAUSE SYSTEM CAUSE TO NPRDS TO NPRDS SUPPLEMENTAL REPORT EXPECTED (14) MONTH DAY YEAR EXPECTED

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

(If yes, complete EXPECTED SUBMISSION DATE).

On July 31, 1995, North Atlantic Energy Service Corporation (North Atlantic) failed to comply with the ACTION requirements in Technical Specification 3.3.1 "Reactor Trip Instrumentation", Table 3.3-1 ACTION 2 for an inoperable Power Range Nuclear Instrumentation (NI) channel. The channel was not placed in the tripped condition within the 6 hours required by Technical Specifications during the performance of a CHANNEL CALIBRATION of channel N41. This failure to comply with an ACTION statement was identified on August 14, 1995 and is being reported pursuant to 10 CFR 50.73 (a)(2)(i)(B).

NO

SUBMISSION

DATE (15)

10

95

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The root cause of this event was determined to be inadequate job scoping of the NI calibration work package. A contributing cause was determined to be inadequate communications between the control board operator and the technician performing the calibration.

Corrective actions taken were to immediately cease all Technical Specification-related work which was moved from the outage to a non-outage period.

# NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (5-92)

APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95

# LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### DESCRIPTION OF EVENT

On July 31, 1995, North Atlantic Energy Service Corporation (North Atlantic) failed to comply with the ACTION requirements in Technical Specification 3.3.1 "Reactor Trip Instrumentation", Table 3.3-1, ACTION 2 for an inoperable Power Range Nuclear Instrumentation (NI) channel. The channel was not placed in the tripped condition within the 6 hours required by Technical Specifications during the performance of a CHANNEL CALIBRATION of channel N41.

Technical Specification 3.3.1, Reactor Trip Instrumentation, Table 3.3-1 ACTION 2 reads as follows:

"With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

- The inoperable Channel is placed in the tripped condition within 6 hours,
- b. The Minimum Channels OPERABLE requirement is met; however the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1, and
- c. Either, THERMAL POWER is restricted to less than or equal to 75% of RATED THERMAL POWER and the Power Range Neutron Flux Trip Setpoint is reduced to less than or equal to 85% of RATED THERMAL POWER within 4 hours; or, the QUADRANT POWER TILT RATIO is monitored at least once per 12 hours per Specification 4.2.4.2."

Technical Specification Surveillance Requirement 4.3.1.1 for Reactor Trip Instrumentation requires a CHANNEL CALIBRATION on each NI Power Range channel at least once per 18 months. These surveillances have routinely been performed during refueling outages. However, the NI calibration procedure and the associated Repetitive Task Sheet (RTS) were reviewed to determine if there were any procedural or operational restrictions that would prevent these surveillances from being performed in MODE 1. The review determined that the surveillances did not meet the definition for "On-line Maintenance" as established in the North Atlantic Management Manual (NAMM). When a task is deemed as "On-line Maintenance" the review process includes risk significance considerations and unique planning, scheduling and implementation requirements. The NI CHANNEL CALIBRATION was not categorized as "On-line Maintenance" and the work was scheduled through the normal planning and scheduling process.

The Power Range Channel N41 Calibration Procedure, provides the necessary instructions required to perform a calibration of NI Power Range Channel N41. The associated Repetitive Task Sheet (RTS) documents the necessary work authorizations and test data. The RTS listed the "equipment out-of-service time", or the time required to perform the surveillance, as eight hours. Actual performance of the calibration procedure varies and without encountering any

#### APPROVED BY OMB NO. 3150-0104 U.S. NUCLEAR REGULATORY COMMISSION NRC FORM 366A EXPIRES 5/31/95 (5-92)ESTIMATED BURDEN PER RESPONSE TO COMPLY THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN LICENSEE EVENT REPORT (LER) THE INFORMATION AND RECORDS MANAGEMENT (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF TEXT CONTINUATION 0001, AND TO THE PAPERWORK (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, DC 20503 WASHINGTON, DOCKET NUMBER (2) PAGE (3) LER NUMBER (6) FACILITY NAME (1) SEQUENTIAL REVISION YEAR NUMBER NUMBER 3 OF 4 Seabrook Station 05000443 95 05

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

problems the surveillance can normally be performed within 4 to 5 hours. The initial steps of the channel calibration procedure directs the associated Nuclear Instrumentation bistables for N41 to be tripped. However, it is necessary to have the bistable energized during the performance of certain sections of the NI channel calibration. These steps are performed later in the procedure near the end of the 6 hour ACTION requirement. In addition, the Power Range Saturation Curve Verification procedure was scheduled at the same time because it required the plant be in MODE 1.

The onshift operating crew entered the required ACTION statements at 1200 on July 31, 1995. The control board operators acknowledged the requirement to trip the bistables within 6 hours. A shift change occurred at approximately 1430 hours and as a part of the shift turnover process the offgoing operator relayed the requirement to verify the bistable tripped by 1800 hours that evening. Shortly before 1800 hours, the onshift control board operator discussed the ACTION requirement with the Instrumentation Technician performing the calibration. Due to a communication error it was believed that the bistables were in the tripped position. The channel was subsequently returned to service at approximately 2130 hours. This resulted in a violation of Technical Specification 3.3.1, Table 3.3-1 Functional Units 2, 3, and 4 ACTION 2 requirements for approximately 3.5 hours. The failure to comply with the ACTION requirement was identified on August 14, 1995 and is being reported pursuant to 10 CFR 50.73(a)(2)(i)(B).

# SAFETY CONSEQUENCES

There are no adverse safety consequences as a result of this event. Sufficient redundancy exists within the Reactor Protection System to ensure a reactor trip would occur in the event of a positive reactivity excursion. When a channel is placed in trip it results in a partial tripped condition requiring only 1 of 3 logic for actuation of the trip circuitry. The bistables were intermittently tripped throughout the performance of the procedure and beyond the 6 hour requirement. It is reasonable to assume the reactor would have been protected due to the inherent redundancy of the Reactor Protection System.

### ROOT CAUSE

The preliminary root cause of this event has been determined to be inadequate job scoping of the NI calibration work package. The planning and scheduling work package review did not recognize that the NI Power Range calibration and saturation curve verification was a time critical evolution when performed as on-line maintenance. The surveillance cannot be performed without having specific bistables tripped and others in normal. It should have been stressed that this surveillance required a 6 hour completion time for compliance with Technical Specification ACTIONS.

A contributing cause was determined to be inadequate communications between the control board operator and the technician performing the calibration.

The evaluation of this event is still in progress. Additional causes or corrective actions, if identified, will be reported in a supplement to this LER. It is anticipated that the supplement, if required, will be submitted by October 15, 1995.

# NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (5-92)

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LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

# SIMILAR EVENTS

This is the first event at Seabrook Station where the Technical Specification actions were not taken during the performance of work which was moved from the outage to a non-outage period.

# CORRECTIVE ACTIONS

As a result of this event, the Assistant Station Director halted all Technical Specification related work which was moved from the outage to a non-outage period. This work was allowed to recommence with the following controls:

- Technical Specification related work was highlighted, and identified as such in the Plan of Day meeting.
- Technical Specification related work scheduled for a specified period is discussed with the scheduled crew prior to commencement.

# PLANT CONDITIONS

At the time of this event the plant was in MODE 1 at 100% power, Reactor Coolant System [AB] temperature at 586.5° Fahrenheit and pressure of 2235 psig.