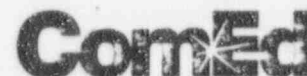


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LWP-95-077

September 7, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Reference: Quad Cities Nuclear Station
Docket Number 50-265, DPR-30, Unit Two

Subject: Licensee Event Report (LER) 265/94-008 Supplemental
Information.

As stated in LER 265/94-008, supplemental information is being provided and is enclosed as Attachment 1. This information constitutes revision 01 to the original LER documentation.

Attachment 2 is a reproduction of the original text of LER 265/94-008.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii)(B), "Any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded or that resulted in the nuclear plant being in a condition that was outside the design basis of the plant."

If there are any questions or comments concerning this letter, please refer them to Nick Chrissotimos, Regulatory Assurance Administrator at 309-654-2241, ext. 3100.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD CITIES NUCLEAR STATION

D. B. Cook for

L.W. Pearce
Station Manager

Attachment 1- LER Supplemental Information
Attachment 2- LER 265/94-008 (copy)

cc: J. Schrage
C. Miller
INPO Records Center
NRC Region III

STMGR107795.LWP

COMMONWEALTH EDISON COMPANY

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SUBJECT: Supplemental Report to LER 265/94-008

BACKGROUND

LER 265/94-008 dealt with the failure of the inboard and outboard Reactor Recirculation sample isolation valves (AO 2-220-44 and AO 2-220-45) to close during quarterly surveillance testing.

The following commitments were made in LER 2-94-008:

- the stroking of valve 2-220-45 at varying frequencies to determine the impact of valve exercising on valve performance (note, valve 2-220-44 was to be maintained closed for isolation purposes);
- the disassembly and inspection of valves 2-220-44 and 2-220-45 and their valve operators at the next outage of sufficient duration; and
- the submittal of a supplemental report upon completion of the investigation.

Supplemental Report:

The disassembly of valves 2-220-44 and 2-220-45 were performed during Q2R13 under NWRs Q15877 and Q11664, respectively. The inspection on 2-220-44 revealed a scratched stem in the area of the packing gland base. The scratch indicates that higher than normal friction loads exist in the valve. During reassembly of this valve all burrs were removed from the valve packing gland base which could have attributed to the scratch. It is unknown how long the valve has been operating with the high stem loads or whether the frictional loads contributed to the stroking failures. No problems were identified during the disassembly and inspection of the 2-220-45 valve.

Discussions with ASCO revealed that some valves have shown a tendency to "stick" under certain conditions where contaminants may enter the valve internals. Microscopic inspections of failed solenoid valves, by ASCO, have identified traces of substances that have adhesive characteristics. However ASCO has not been able to determine the time that is required for this phenomenon to impact the valve movement. ASCO is currently performing tests to determine the origin of these substances. ASCO recommends increasing the stroke testing frequency, as determined by the user, to prevent this from impacting valve performance.

FURTHER ACTIONS:

Based on the results of the stroking frequency tests a new procedure, QCOS 202-13, Monthly Testing of Reactor Recirc System Air Operated Valves has been implemented. This monthly stroking frequency is consistent with that specified by the vendor to ensure proper valve performance of valves 220-44 and 220-45 on Units 1 and 2.