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GEORGIA POWER
POWER GENERATION DEPARTMENT
VOGTLE ELECTRIC GENERATING PLANT

TRAINING LESSON PLAN

2-1324

TITLE:	EPIP OVERVIEW	MUMBER:	LO-LF-40101-11-C
PROGRAMI	LICENSED OPERATOR TRAINING	REVISION	11
SME:	L. RAY	DATE:	10/20/89
APPROVED:	that a train	DATE:	12/1/89

## INSTRUCTOR GUIDELINES:

- I. LESSON FORMAT
  - A. Lecture with Visual Aids
- II. MATERIALS NEEDED
  - A. White Board with Markers
  - B. Overhead Projector
  - C. Transparencies

## III. EVALUATION

A. Written or Oral Exam in conjunction with other Lesson Plans

#### IV. REMARKS

- A. Training in Emergency Plan is a requirement per IEN 84.042 and Procedure 60602-C. Responsibilities and care damage assessment for SRO's is required per 60601-C. Sec 4.1.3.3.0(2) Rev 2
- 3. Performance-based instructional units (IUs) are attached to the lesson plan as student handouts. After the lecture on EPIFs, the student should be given adequate self-study time for the IUs. The instructor should direct self-study activities and be available to answer questions that may arise concerning the FU material. After self-study, the student will perform, simulate, observe, or discuss (as identified on the cluster signoff criteria list) the task covered in the instructional unit in the presence of an evaluator.

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Licensed Operator Objectives for this lesson plan can be found in the Licensed Operator System Master Plan Section 2.3 (Qualification Signoff Criteria)

Rev 4 Cluster 40 SRO EMERGENCIES

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#### 1. PLANT VUGTLE IROCEDURES:

91001-C, EMERGENCY CLASSIFICATION AND IMPLEMENTING INSTRUCTIONS

91002-C, EMPRGENCY NOTIFICATIONS

91101-C, EMERGENCY RESPONSE ORGANIZATION

11102-C, DUTIES OF THE EMERGENCY DIRECTOR

91201-C, ACTIVATION AND OPERATION OF THE TECHNICAL SUPPORT CENTER

91202-C, ACTIVATION AND OPERATION OF THE OPERATIONS SUPPORT

91203-C, ACTIVATION AND OPERATION OF THE EMTRGENCY OPERATION FACILITY

91204-C, EMERGENCY RESPONSE COMMUNICATIONS

91301-C, EMERGENCY EXPOSURE QUIDELINES

91304-C, COMPUTERIZED AND MANUAL BACKUP METHODS FOR RELEASE RATE AND DOSE CALCULATIONS

91305-C, PROTECTIVE ACTION GUIDELINES

91401-C, ASSEMBLY AND ACCOUNTABILITY

9 402-C, SEARCH AND RESCUE

91403-C, SITE EVACUATION 91501-C, RECOVERY

91501 C, CORE DAMAGE ASSESSMENT

91503-C, CONTROL ROOM INSTRUMENTATION OUTPUT FOR ASSESSMENT OF CORE DAMAGE

# 2. COMMITMENTS AND OTHER REQUIREMENTS:

IEN 84.042

FF 89.007 FEEDBACK FROM E DRILL FF 89.008 FLEDBAC? FFOM E DRILL

## 3. STUDENT HANDOUTS

LO-HO-40101-001

## 4. TRANSPARENCIES:

LO-TP-40101-001 FOUR EMERGENCY CLASSES

LO-TP-40101-002 NOTIFICATION OF UNUSUAL EVENT

LO-TP-40101-003 ALERT

LO-TP-40101-004 SITE AREA EMERGENCY

LO-TP-40101-005 GENERAL EMERGENCY

LO-TP-40101-006 EMERGENCY RESPONSE ORGANIZATION

LO-TP-40101-007 ON-SHIFT EMERGENCY RESPONSE ORGANIZATION

LO-TP-40101-008 ACTIVATION OF EMERGENCY RESPONSE ORGANIZATION

LO-TP-40101-009 TSC MANAGER

LO-TP-40101-010 OSC MANAGER

LO-TP-40101-011 EOF MANAGER

LO-TP-40101-012 ALERT SITE BOUNDARY RADIATION LEVELS

LO-TF-40101-013 SITE AREA BOUNDARY RADIATION LEVELS

LO-TP-40101-014 GENERAL EMERGENCY SITE BOUNDARY RADIATION LEVELS

LO-TP-40101-015 CSF'S MONITORED TO DETERMINE FISSION PRODUCT BARRIER
BREACH

LO-TP-40101-016 CSF'S MONITORED TO DETERMINE FISSION PRODUCT BARRIER
STATUS

LO-TP-40101-017 EMERGENCY EXPOSURE LIMITS
LO-TP-40101-018 CASITE PROTECTIVE ACTIONS
LO-TP-40101-029 OFFSITE PROTECTIVE ACTIONS
LO-TP-40101-020 RECOVERY ORGANIZATION

LO-TP-40101-021 FUEL DAMAGE VERSUS ELEVATED FUEL ROD TEMPERATURES
LO-TP-40101-022 EMERGENCY CLASS DETERMINATION SCENARIO #1
LO-TP-40101-023 EMERGENCY CLASS DETERMINATION SCENARIO #2
LO-TP-40101-024 EMERGENCY CLASS DETERMINATION SCENARIO #3
LO-TP-40101-025 EMERGENCY CLASS DETERMINATION SCENARIO #4

## 5. INSTRUCTIONAL UNITS

LO-IU-40101-002 MAKE EMERGENCY CLASSIFICATION
LC-IU-40101-002 MAKE EMERGENCY NOTIFICATION
LC-IU-40101-003 PERFORM MANUAL OFFSITE DOSE CALCULATIONS
LO-IU-40101-004 IMPLEMENT EMERGENCY PROTECTIVE ACTIONS AND
RECOMMENDATIONS
LO-IU-40101-005 IMPLEMENT SITE EVACUATIONS
LO-IU-40101-006 IMPLEMENT RECOVERY ACTIONS
LO-IU-40101-007 ASSEMBLE AND DISPATCH FIRE BRIGADE

#### I. INTRODUCTION

- A. This lesson will provide the student with an indoctrination into EPIP's, with particular emphasis on EPIP's that the licensed operator is responsible for or directly interfaces with in the event of an emergency.
- B. Present Lesson Objectives

## II. PRESENTATION

	1460.00		
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- 1. These are four classifications of emergencies In order of severity (least to worst) they 1918
  - a. Notification of Unusual Event (NUE)
  - b. Alert
  - c. Lite Area Emergency
  - d. General Emergency

# 2. Summary of Classes

a. Notification of Unusual Event

Events which indicate a potential degradation of the level of safety of the plant.

b. Alert

Events which involve an actual or potential substantial degradation of the level of safety of the plant.

c. Site Area Emergency

Events which involve actual or likely LO-TP-40101-004 major failures of plant functions needed for protection of the public

d. General Emergency

Events which involve actual or imminent LO-TP-40101-005 substantial core degradation or melting with the potential for loss of

Start: IEN B4.042

Objective 2

LO-TP-40101-001

Handout #1 Page 2

LO-TP-40101-002

LO-TP-40101-003

Containment integrity. An evacuation of off-site personnel is implemented for this emergency.

- 3. The Emergency Plan Implementing Procedure (EPIP's) are a group of procedures that implement each aspect of the emergency from initiation to recovery from the emergency
- 4. The Emergency Plan is an agreement between GPC, state, local, and federal officials and specifies the actions of each in the event of an emergency. The EPIP's implement GPC's responsibility with respect to the Emergency Plan.
- 5. The Emergoncy Director (ED) has the overall authority and responsibility for the implementation of the EPIP's.

Objective 1
The OSOS initially fills the position of ED if the General Manager is not on-site

 When using EPIP's, cross through or cross out steps in checklist as they are completed

FT-89.007

6. The Key EPIP's are:

Objective 3

a. Duties of the Emergency Director (91102-C) - provides instructions to the FD in fulfilling his responsibility for overall coordination and direction of the Emergency Response Organization

Handout #1 Pages 3 and 4

b. Emergency Classification (91001-C) provides instructions for the classification of off-normal events into one of four emergency classification levels. It also provides initial implementing instructions to the ED

2D responsibility

c. Emergency Notification (91002-C) provides instructions to complete
initial and follow up notification to
GPC cn-site and off-site emergency
response personnel and to Georgia,
South Carolina, local counties, and
NRC officials.

ED makes Emerg. Motification

d. Site Evacuation (91403-C) provides instructions for the early dismissal or evacuation of non-essential

ED determines need for evacuation and orders evacuation. personnel from the plant site in the event of an emergency

- e. Assembly and Accountability (91401-C) provides instructions for the assembly
  of non-essential personnel to their
  designated assembly area and the
  accounting of personnel to identify
  those who may be missing
  - Ops personnel should badge into control room or call control room on alert or higher classification
  - 2) Personnel receiving call-in should ask for badge number (not ACAD)
- f. Protective Action Guidelines (91305-C)

   provides instructions for the on-site and off-site protective actions alternatives to ensure plant and off-site personnel safety in the event of an emergency

- g. Recovery (91501-C) provides instructions for determining
  when the emergency condition will be
  terminated and the recovery phase
  initiated. It also describes the process
  used to notify and activate the recovery
  organization for immediate and long term
  recovery operations.
- h. Core Damage Assessment (91502-c) provides a method to classify and estimate
  the extent of core damage through core
  fission product release measurements,
  reactor vessel level indications, and
  core exit thermocouple temperatures
  along with other auxiliary readings
- 7. Emergency Response Organization (ERO)
  - a. The Emergency Response Organization is activated by the ED.

ED orders assembly and accountability

Assembly area is the Admin, Bldg, for VEGP personnel

FF 89.008

ED implements onsite Prot. Actions and recommends off-site Prot. Act. Prot. Actions On-site Early Dismissal Site Evacuation

Off-site Seek Shelter Evacuation

ED determines recovery criteria and mobilizes the Recovery Organization

Chemistry is responsible for assessment. Operation in part, provides data to Chemistry per 91503-C, "Control Instrumentation Data Record for Core Damage Assessment".

- b. The Emergency Response Organization is made up of 4 groups.
- c. Each group has defined (by the EPIP's) responsibilities
- d. The 4 groups are:

LO-TP-40101-006

- The Technical Support Center (TSC)
  Group
- 2) The Emergency Off-site Facility (EO,\*) Group
- 3) The Operations Support Center (OSC) Group
- 4) The Control Room
- e. On back whifts and for the time frame that it takes to activate the ERO the On Shift Emergency Response Organization (OSERO) is activated (members shown data)

LO-TP-40101-007

f. The Activation of the ERO's for emergency classes is:

LO-TP-40101-008

- 1) Notification of Unusual Event OSERO
- 2) Alert initially OSERO; then TSC, OSC, Control Room
- 3) Site area and General Emergency initially OSERO: then TSC, EOF, OSC,
  Control Room

SCF is activated for Site and General Emergencies

- g. For both the ERO and the OSERO, the ED has functional authority over all the groups
- b. The ED when the OSERO is in effect is the OSOS
- i. The ED for the ERO is the Senior VP Nuclear Operations or the Genera. Manager
- j. Each ERO group is headed by a Manager
- 8. Technical Support Center Group
  - a. The ED activates the TSC

	b.	The	TRO	Manager	heads t	c has	TSC	group
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LO-TP-40101-009

- 1) Primary Plant Manager
- Alternates General Manager, Manager Operations, Operations Superintendents
- c. The TSC is a located adjacent to the control room and is equipped with instrumentation that allows monitoring of many plant parameters
- d. The TSC group is made up of plant supervisors and plant staff
- e. The functions of the TSC group are:
  - Provide in plant radiological assessment and protective actions recommendations
  - Engineering and technical analysis for control room support
  - 3) Liaison between TSC and CR
  - 4) Coordination of emergency maintenance
  - 5) Direction of post accident and plant chemistry sampling and analysis, evaluation of chemistry data, core damage assessment
  - Planning and implementation of Logistical support
  - 7) Assess control and accountability in TSC
  - 8) The TSC upper limit of habitability is
    - a) Whole body dose 100 mr/h in TSC
    - b) Todine activity 2.7 E-07 uCi/cc in TSC
  - 9) The TSC Manager relocates to the backup location of the Control room
- 9. Operations Support Center

Handout #1 Page 5 Objective 4 HF Supervisor

Engineering Supv.

Operations Supv.

Maincenance Supv.

Chemistry Supv.

TSC Support Coord.
(Document Control)
Supv.
TSC Security Coord
(Nuclear Shift
Supv)

4

a. The ED orders activation of the OSC

Objective 49

- b. The OSC is located on the second floor of the Maintenance Building
- The OSC Manager is the Maint. Manager. The alternates are the Maint. Supervisor(2), Superintendent(s)

LO-TP-40101-010

- d. The OSC serves mainly as a staging area for the deployment of Radiological Emergency Teams (RET's)
- e. The functions of the OSC are:

Objective 4 Headout #1, Page 6

- Serve as the assembly and staging area for personnel pooled for emergency response
- Respond to request from the ED and the TSC concerning deployment of RRT's
- Management of emergency equipment and supplies
- 4) Coordination of movement of personnel on-site

## f. Function of RET's

 Search and Rescue Team - performe emergency search and rescue of individuals that are unaccounted for or disabled Objective 5
Handout #1, Page 7
Per 91402,
"Search and
Rescue"

2) First Aid Team - performs first aid for radiological and non-radiological injuries and personnel decontamination as required

Per 91306 "Contaminated Injury"

3) Damage Assessment/Control Team - a selected group of personnel who perform damage assessment and control activities as needed

Per 91306
"Contamination
Monitoring and
Decontamination"

4) Repair and Modification Team - a selected group of personnel who perform repair work and modify systems, equipment, etc., at needed

Fer 91306
"Contamination
Monitoring and
Decontamination"

5) In Plant Monitoring Team - performe

Per 91302-0

in plant sampling and surveys. The HP Supv. uses this information when Protective Action recommendations to the ED

"In Plant Sampling and Surveys"

- 5) Back-Up Fire Brigade Back up support to the normal shift fire brigade
- g. The OSC upper limit of habitability is:

Objective 50

- 1) Whole Dody dose rate of 100 mr/hr in OSC
- 2) Iodine activity of 2.7 E-07 uCi/cc in OSC
- h. The OSC Manager then makes the decision to relocate to the backup location of the TSC
- 10. Emergency Operations facility Group
  - a. The ED orders activation of the EOF

Objective 49

- b. The primary EOF is located at the VEGP Training Center
- c. The EOF Manager is the Outage and Planning Manager with Plant Training and Emergency Preparedness Manager, Health Physics Superintendent backups

LO-TP-40101-011 CLDY Activated on a Site and General Emerg.

- d. The ECF serves mainly as the liaison and coordinator between GPC and off-site authorities
- e. The functions of the ROF group are:
  - Direction and control of GPC off-site emergency response
  - 2) Assistance to the ED and supervision of ROF
  - Communication of radiological information to state and local emergency response agencies
  - 4) Support of initial activaties associated with planning for re-entry or recovery operation

Objective 4 Handous #1, Page 8 Per "Duties of EOF Manager"

EOF Manager

EOF Manager

EOF Manager

- 5) Access control for BOF
- 6) Processing of personnel who require authorization to enter site

EOF Security Coordination

 Requesting assistance from local law authorities

Security Coordinator

8) Performance of off-site dose calculation

Dose Assessment Manager

9) Direction of Field Monitoring Teams

Dose Ass. Manager Via Mon. Team Coordinator

10) Coordination with state and Federal Groups performing radiological assessment

Dose Ass. Manager

11) Development of protective action recommendations

Dose Ass. Manager

12) Assumption of logistics support functions from the TSC

EOF Support

13) Providing press releases when authorized by the ED

Public Information Manager

f. The EOF upper limit of habitablity is:

Objective 50

- 1) 100 mr/hr wholn body
- 2) Iodine activity 2.7 E-07 uCi/cc
- g. When inhabitable, the EOF manager will relocate the EOF to the back up location at the GPC district office in Waynesboro

11. Recovery Organization

Objective 4

- a. Recovery organization staff shall be responsible for:
  - Assuming assigned positions and ensuring that support staff are available and properly briefed
  - Ensuring that relief personnel are fully briefed prior to relinquishing their responsibilities
- 12. Health Physics Group

- a. The HP Group provides the following key functions during an emergency:
  - 1) Performs off-site dose calculations
  - 2) Discusses on-site and in plant radiological conditions
  - Provides on-site and off-site radiation protection measures
- 13. Chemistry Group

Objective 4

- a. The Chemistry Group provides the following key function during an emergency
  - 1) Assess core damage from accident
  - 2) Evaluates in plant chemistry analysis
  - 3) Operates PASS
- B. Duties of the Emergency Director (91102-C)
  - The ED position is normally filled by the Vice President, Nuclear or the General Managar, Nuclear Plant. The first alternate is the Assistant General Manager Plant Operations. The second alternate is the Assistant General Manager Plant Support. The third alternate is the Manager, Operations

Objective 7

Operations provides data to

Chemistry

for damage

- The OSOS will initially fill the position if neither of the primaries are on-site
- Objective 6
- 3. The Emergency Director has the following responsibilities which cannot be delegated
- Objective 8 Handout #1, Page 9
- a. Classifying and declaring the emergency, including upgrading, downgrading, or termination
- EPIP "Emergency Classification and Implementing Instructions" 91305-C
- Recommending protective actions to off-site authorities and content of messages
- "Protective Action Guidelines" 91305-C
- Deciding to evacuate non-essential personnel from the site at the Alert
- "Site Evacuation" 91403-C

classification level

d. Authorizing personnel radiation exposures in excess of 10CFR20 limits, if necessary

"Emergency Exposure"

- e. Deciding to request assistance from federal support groups
- f. Deciding to notify off-site authorities responsible for emergency measures

"Emergency Notifications" 91002-C

- 4. Other ED responsibilities are:
  - Maintaining communications with off-site authorities regarding all aspects of emergency response

"Emergency Response Communications" 91204-0

- Provide overall direction for management of procurement of site-needed materials, equipment, and supplies, documentation, accountability, and security function
- of the emergency organization; including emergency response facility activation

Activation of TSC, EOF, OSC 91201, 202, 203-C

- d. Coordinating and directing VEGP emergency operations
- Response Organization staffing, as needed. The ED is the only individual authorized to approve modifications to EPIPs

Objective 9

- f. Coordinating NRC activities to reduce duplication of effort and reduce the impact on plant staff during the emergency situation
- g. The OSOS when acting as the ED shall either be in the control room or the TSC

The primary ED is normally in the TSC (Alert) or EOF (Site or General)

- h. For any emergency classification of, alert or higher, the OSOS must be relieved by the primary ED as soon as possible
- C. Emergency Classification (91001-c)

- The OSOS is responsible for the initial classification of an emergency and filling the position of ED until relieved
  - a. Mark through or cross out steps in checklists as they are completed

FF 89.007

 When classifying an emergency, the following categories are considered as the criteria for classification

Objective 12 Handout #1, Fage 10

- a. Plant Power
- b. Technical Specifications
- c. Radioactivity
- d. Plant Systems Status
- e. Natural Phenomenon
- f. Man-Made Hazards
- g. Security
- h. Fission Product Barriers
- i. Shutdown Systems
- j. Other a category or emergency not specifically in which a condition threaten the safety of the plant or public
- 3. Emergency Classification
  - Notification of Unusual Event Classifications
    - 1) Plant Power
      - A) Loss of on-site or off-site A.C. power
    - 2) Technical Specifications
      - a) Loss of ESF, PERMS monitor or Effluent Air Flow Inst. requiring Plant Shutdown per Tech Specs
      - b) Exceeding RCS leak rates per

Damage to Spent Fuel in containment or spent fuel area

Objective 12 Handout #1, Page 11 Tech Spece

- Exceeding RCS activity per Technical Specifications
- d) Any Tech Spec. safety limit violation

## 3) Radioactivity

- Transportation off-site of a contaminated injured victim
- b) Gross failed fuel monitor exceeds 2E+4 cpm above normal or confirmed > Tech Spec by lab analysis
- Radioactive effluent exceeds Tech Specs

Liquid and gaseous limits as specified in the Off-sits Dose Calculation Manual (ODCM)

# 4) Plant Systems

- a) Rapid depressurization of the secondary system
- Turbine rotating component failure causing rapid plant shutdown
- c) ECCS discharge to reactor vessel

Any SI

- Reactor protection actuation without subcriticality
- Pailure of a safety or relief valve to reclose in a safety reluted system

### 5) Natural Phenomenon

- a) Any earthquake felt or dofected on-site
- b) Tornado or hurricane on-site
- c) Fire in the plant lasting longer than 10 minutes
- 6) Man-made Hazards

- a) Aircraft crash on-site
- b) Train derailment on-site
- c) On-site explosion
- d) On-site toxic gas release
- 7) Security
  - a) Unusual aircraft activity overhead
  - Security threat or attempted sabotage
- 8) Fission Product Barrier
  - a) Not considered for this classification
- 9) Shutdown Systems
  - a) Not considered for this classification
- 10) Other
  - a) Loss of all meteorological data
  - b) Loss of both site telephone ( d ENN switches
- b. Alert ,
  - 1) Plant Power
    - a) Lose of all off-site and on-rive A.C. power
    - b) Loss of all vital D.C. power
  - 2) Technical Specifications
    - a) Not considered for this classification
  - 3) Radioactivity

LO-TP-40101-012

a) Site boundary dose rate greater than .5 mrem/hr whole body or 1.5 mrem/h thyroid b) Radiation level or airborne contamination which indicates degradation in the control of radioactive material

An increase of 1000 cimes normal

## 4) Plant Systems

- Turbine failure causing casing penetration and damage to safety systems
- b) Evacuation of control room anticipated or required with control of shutdown system established from shutdown panels

#### 5) Natural Phenomenon

- a) Earthquake greater than .12 g.
- b) Subtained hurricane winds sustained greater than 90 mph
- Tornado striking plant structure or the protected area

#### 6) Man-Made Hazards

- a) Aircraft crash or missile impact inside the protected area
- Explosion damage affecting safety system
- Uncontrolled toxic or flammable gases entering the protected area

#### 7) Security

- a) Ongoing security comprise which results in intruders within the Protocted Area
- 8) Fission Product Barrier
  - a) Breach or challenge of 1 of 3 fission product barriers
- 9) Shatdown Systems

Example: clad damage as determined from CFMS

Example: Loss of all RHR

- a) Complete loss of any function which results in inability to maintain cold shutdown
- 10) Other
  - a) Precautionary activation of the TSC

If the ED activates the TSC as a precaution, Emer. classification must be an Alert

- c. Site Area Emergency
  - 1) Plant Power
    - AC power for more than 15 minutes
    - b) Loss of all vital DC power for more than 15 minutes
  - 2) Technical Specifications
    - a) None considered for this classification
  - 3) Radioactivity
    - a) Site boundary dose rates projected or measured greater than 50 mrem/h whole body

LO-TP-40101-013

- b) Greater than 250 mrem/h thyroid
- 4) Plant Systems
  - Evacuation of the control room and control of shutdown systems not established from shutdown panels in 15 minutes
- 5) Natural Phenomenon
  - a) Plant not in cold shutdown and earthquake greater than .2g Qr custained winds greater than 100 mph.
  - b) Fire resulting in the loss or encassive degradation of ony safety-related function
- 6) Man-Made Hazards

- a) Flant not in cold shutdown and:
  - Aircraft crash affecting vital structures by impact or fire
  - (2) Severe damage to safe shutdown equipment from missiles or explosion
  - (3) Uncontrolled flammable gases entering the vital areas
  - (4) Uncontrolled toxic gases in the vital area restricting access causing a safety problem
- 7) Security
  - a Ongoing security compromise which results in intruders in the vital area
- 8) Fission Product barrier
  - a) Breach or challenge of 2 of 3 fission product barriers
- Clad and RCS piping
- Known LOCA greater than makeup capacity
- 10) Shutdown Systems
  - a) Complete loss of any function which results in inability to maintain hot shutdown
- 11) Other
  - Major damage to spent fuel in the containment or fuel handling building
- d. General Emergency
  - 1) Plant Power
    - None considered for this classification
  - 2) Technical Specifications
    - a) None considered for this

## classification

3) Radinas vity

LO-TP-40101-014

- a) Site boundary profacted or actual dose rate greater than 1 rem/h whole body or 5 rem/h thyroid
- 4) Plant Systems
  - a) Loss of physical control of the plant
- 5) Natural Phenomenon
  - a) None considered for this classification
- 6) Man-Made Hazards
  - None considered for this classification
- 7) Security
  - a) Loss of physical control of plant
- 8) Fission Product barrier
  - a) Breach or challenge of 3 of 3 fission product barriers

Clad, RCS and Ctmt.

- 9) Shutdown Systems
  - None considered for this classification
- 10) Other
  - a) None considered for this classification
- D. Use of Figures and Data Sheets in EPIP 91001-C
  - 1. Data Sheet 1 is used to document classification determination
  - Fission product barrier status is first determined by the operators

Objective 11

The barriers are:

- a. Puel pellet matrix and cladding
- b. RCS pressure boundary
- c. Containment system
- 3. One way the operator can determine breach or challengs to the barriers is by monitoring the plants Critical Safety Functions (CSF's) via the SPDS/ERF computer or 19200-C. The CSF's are:

LO-TP-40101-015 Objective 11

- a. Reactivity (subcriticality)
- b. Core Cooling
- c. Heat Sink
- d. RCS Integrity
- e. Containment Integrity
- To determine fission product barrier status;
  - a. Fuel/cladding Integrity a red or orange terminus from reactivity, or core cooling indicates a breach. A red terminus from heat sink indicates a breach

LO-TP-40101-016 Objective 10/11

- RCS a red or orange terminus from core cooling or RCS integrity indicates a breach
- Containment integrity a red or orange terminus from containment integrity indicates a breach
- 5. After the fission product barrier status is determined, Figure 4 from 91001-C is used to determine the emergency classification and if other factors should be considered in the classification
- A checklist for each classification exist in 91001-C that guides the ED in his initial actions and notifications
- 7. An emergency is downgraded as follows:
  - a. The ED determines that conditions are favorable for downgrading an emergency

- b. Notification will be made to reate, local and federal officials notifying them that the emergency has been downgraded
- c. Complete all actions that are necessary for that emergency classification

# E. Emergency Notifications (91002)

- 1. This procedure provides instructions to complete on-site and off-site emergency notifications
- 2. The following is a list of responsibilities and actions:
  - a. The Emergency Director is responsible for approving initial and follow-up messages to the federal, state and local off-site authorities as well as designating Communicator/Recorders to transmit these messages. (91002-C)
  - b. The Emergency Director is responsible for the decision to notify federal, state, and local off-site auchorities that an emergency has been declared and to recommend any protective actions (91002-0)

c. A communicator shall start immediately to ED designates inform both on-site and off-site personnel communicator with required limits

d. Initial notification of Georgia, Burke County, SRS, South Carolina, Aiken County, Barnwell County, and Allendale County off-eite authorities shall be accomplished within 15 minutes of the declaration of an emergency or an upgrade to a more severe emergency classification level (91002)

Objective 20

Objective 16

Objective 17b

Objective 15

1) The primary means for accomplishing thase notifications is the Emergency Notification Network (ENN). (91002-C) 2) If the SNN is not working, cremercial

telaphones should be used to make the notifications for Georgia. A backup ENN phone is used as backup Objective 22 for South Carolina

- If the commercial telephones are not working, radios should be used
- 4) Once a working line of communication is established, the notifications must be accomplished in accordance WITH the checklists found in EPIP 91002-C
- e. The NRC must be notified as soon as possible within 60 minutes of the declaration of the emergency or whenever an emergency is upgraded to a more severe emergency classification (91002-C sec. 4.2)

Objective 16 Objective 17a

 The primary means for accomplishing this notification is the Emergency Notification System (ENS). (91002-C)

Red phone in control room and TSC Objective 21

2) If the ENS is not working, commercial telephones should be used to make the notification (91002-C)

Objective 23

 Once a working line of communication is established, the notification must be accomplished in accordance with the checklist (91002-c)

Objective 18

e. Follow-up messages must be completed and transmitted to the state and local off-site authorities whenever there is a significant change in plant conditions or at least every 30 minutes (91002-C)

Objective 19

- F. Follow-up messages to the NRC should be accomplished whenever there is any further degradation, any change in one class to another, or termination of the emergency.
- G. Amergency Communications
  - 1. Emergancy Notification Network (ENN)
    - a. Black phone

- Used to notify Georgia and south Carolina state, country and SRP authorities
- The phone is located in the control room, TSC, BOF, and the county and state emergency centers

Objective 22

Objective 22

Objective 21

- d. The ENN network is energized by litting the receiver. All speakers are energized
- e. To speak the button on the hand set must be depressed
- 1. A dedicated two digit phone is the backup ENN for South Carolina. Also, commercial telephones and SRP radio in TSC are backups
- g. The commercial phone system is the backup for Georgia. Another backup is the Burke County radio located in the TSC
- Emergency Notification System (ENS)
  - a. Black phone with red dial face in control room, TSC, EOF, and NKC Resident Inspectors Office
  - b. Used for notification to the NRC Region II in Atlanta
  - c. The phone is activated by lifting the recuiver. The phone at the NRC office will ring
  - d. The backup to the ENS is the Commercial Objective 23 phone system

## 3. Radios

- a. Radio base stations are placed strategically within the plant for security and maintenance
- b. Various other radios in the TSC, control room and other locations are used for communication with on-site Radiological Emergency Teams and off-site authorities

## Site Evacuation

- 1. The site evacuation procedure provides instructions for the early dismissal or evacuation of non-essential personnel including visitors at VEGP in the event of an emergency.
- 2. The Emergency Director is responsible for determining the need for evacuation and

NOTES

selecting evacuation routes

 The Security Coordinator is responsible for conducting the evacuation and directing the security force to assist in various stages of the evacuation

Objective 25

4. A size evacuation is required to protect onsite non-essential personnel from radiation exposure or physical harm

objective 24

- 5. The Emergency Director shall determine the need for evacuation given existing or anticipated conditions, after consulting with:
  - a. H.P. Supervisor
  - b. Dose Assessment Manager
  - c. TSC Manager
  - d. EOF Manager
- 6. The ED has two options:
  - a. Early dismissal at which time all nonessential personnel are sent home early
  - Evacuation of personnel to an assemble site

YEGP Recreation Area primary assembly point; Plant Wilson Secondary

- Evacuation or early dismissal shall be ordered (if feasible) for an emergency classification of Site Area or General
- 8. An early dismissal of non-essential personnel is normally conducted for an Alert
- I. Assembly and Accountability
  - Assembly and accountability is ordered by the FD for any emergency classification of Alert or higher

Objective 27

 The primary assembly point for non-essential personnel is the VEGP recreation facility

Objective 26

 The backup location is Plant Wilson and would be vaid if radiological conditions

e.g., if wind conditions were

did not allow the primary recreation area to be used

toward the recreation area

A. Accountability is required to possibly identify any endangered personnel in the protected area that are unable to leave

"Search and Rescue" used to rescue unaccounted for personnel

5. The Security Coordinator is responsible for the accounting of personnel

Objective 28

- 6. Accountability reports by the Samulty Coordinator must be made to the ED within 30 minutes
- 7. The OSOS and the TSC and OSC manager are responsible for accountability of personnel assigned to the control room, TSC, and OSC
- 8. The accounting of control room, TSC, and OSC shall be made within 30 minutes of the order

## Search and Rescue

1. Search and Rescue procedure provides instructions for the search and rescue of individuals who may be missing or disabled in the protected area.

Objective 29

- 2. The Emergency Director shall be responsible for implementing this procedure by directing the TSC Manager to dispatch a search and rescue team
- a. If the OSC has not been activated, the On-Shift H.P. Foreman shall form the team using available staff
- 3. The OSC manager shall be responsible for: Objective 30

- a. Forming
- b. Briefing
- c. Dispatching the search and rescue team

- 4. The Search and Rescue Team Leader shall be responsible for following:
  - a. Following instructions

- Maintaining Communications with OSC Manager or H.P. Foreman
- 5. Each team must consist of at least two members. Each team member must be familiar with the plant and first aid training

Objective 32

- 6. Exposurer in excess of 10CFR20 limits shall be approved by the Emergency Director
- 7. The Emergency Director shall direct the TSC Manager or HP Foreman to form a search and rescue team as soon as he is made aware of a missing or disabled person
- The following guidelines shall be employed during the rescue effort.
  - a. Protective clothing and respiratory protection may be worn
  - b. Jse of lifelines and respiratory protection for areas that are smoke or steam filler.
  - c. Infurm the control room of projected actions

# K. Emergency Exposure Guidelines

- The purpose of this procedure is to provide instructions and controls for radiation exposures in excess if 10CFR70 occupational limits during emergency conditions
- The Emergency Director has the sole authority for all radiation exposures in excess of 10CFR20 limits

Objective 33

- Personnel authorized to receive exposures in excess of 10CPR20 limits should meet the following criteria:
  - a. Personnel must be fimiliar with the risks of Oxposurs to higher radiation doses which are likely.
  - b. Should have current radiation exposure history and receive a briefing. For exposures greater than 25 rem personnel should be volunteers. Consider volunteers above the age of 45 first.

- c. Females should not be of child bearing age and capability except for urgent lifesaving actions
- Objective 35
- 4. The whole body emergency exposure limits are:

Objective 34

a. 5 Rem for sampling

LO-TP-40101-017

- b. 25 Rem for protective or corrective action
- c. 75 Rem to save a life

## L. Protective Action Guidelines

- Protective actions for on-site and off-site personnel are planned provisions to ensure their safety in the event of an emergency
- The Emergency Director is responsible for implementing on-site protective action
- 3. The ED is responsible for recommending off-site protective actions

Only the states can enforce protective actions

- 4. Protective action recommendations shall be made on the basis of current or projected plant condition and/or calculated or measured dose rates
- 5. The HP Supervisor is responsible for evaluating on-site radiological conditions for making on-situ protective action recommendations to the ED
- The Dose Assessment Manage is responsible for making off-site dose estimates and recommending off-site protective actions
- 7. The on-site protective actions are:
  - a. Non-essential personnel early dismissal or plant evacuation
  - b. Distributing potassium todine (KI) for plant workers
  - Use of additional issumetry, respiratory protection, and protective

LO-TP-40101-018
Objective 36
Per Site
Evacuation EPIP mandatory for Site
Area or General
Emergency
KI is a lodine
absorption blocker
of the thyroid

LO-TP-70101-019

#### clothing

- 8. Off-site Protective Actions are:
  - a. Evacuation of off-site personnel of at least a 2 mile radius
  - b. Further ovacuation of affecting sectors
  - c. Recommend personnel to seek shelter, close windows, secure ventilation systems, etc.

#### M. Recovery

- This procedure provides instructions for determining when the emergency condition will be terminated and the recovery phase initiated
  - a. It also describes the process used to notify and activate the recovery organization as well as its functions and responsibilities, including management of resources for long-term recovery operations
- The Amergency Director directs the notification and activation of the recovery organization. (Consults with V.P. Nuclear Operations).
- 3. The Recovery Manager shall have overall responsibility for restoring the plant to a normal operating configuration. The primary for this position is the VEGP General Manager Nuclear Plant
- Recognizing the following conditions the Emergency Director has decided to terminate the emergency condition:
  - a. Plant radiation levels are stable or decreasing with time
  - b. The affected reactor is in a stable condition and can be maintained indefinitely in a stable condition
  - c. Fire or other similar emergency

Objective 37
Minimum evac. for General Emergen:y based on meteorological information An option if off-site doses do not reach the evacuation threshold

Objective 40

Objective 38 Handout #1, Page 15

conditions to longer constitute a hazard to safety-related systems, equipment, or personnel.

- d. Radioactive releases to the environment have ceased or been controlled to within permissible license limits
- e. Discussions with GPC management, VEGP Emergency Response organization, and off-site authorities do not result in identification of any valid reason for not terminating the emergency
- 5. When the ED has formally terminated the emergency, command is transferred to the Recovery Manage and Recovery Organization
- 6. The Recovery Organization is made up of individuals who manage and direct personnel who:
  - a. Perform required maintenance
  - b. Develops recovery schedule
  - c. Provides HP support during the entire recovery period
- N. Computerized and Manual Backup Method for Release Rate and Dose Calculations
  - The purpose of this procedure is to provide instructions for personnel to estimate radiclogical releases resulting in off-site dones which would require the implementation of protective action recommendations.
  - 2. The OSOS or designee shall initially be responsible for determining release rates and carrying out initial off-site dose calculations
    - a. H.P. Supervisor is responsible if the TSC is activated
    - b. Dose Assessment Manager is responsible if the EUF is activated
  - The primary method of calculation of off-site doses is the VIBRANT computer dose calculation

LO-TP-40101-020 Objective 4

Objective 41a

Objective 41b

- 4. The calculation uses the following input data
- Objective 44 Handout #1, Page 21
- a. Windspeed and direction (primarily from ERF computer)
- Plant radiological and effluent data (primarily from ERF computer)
- c. Direct measurement of effluent path with a portable inst.
- 5. The Output of the VIBRANT calculation is

Objective 45

- a. Plume arrival time
- b. Direction (X/Q)
- c. Whole body doses
- d. Infant thyroid doses
- e. Doses at preset downwind distances

Handout #1, Page 22

6. The manual backup method is used as a backup method and is used with any of the following Fost Accident Accident Monitors (PAM's):

Objective 42
The OSOS will in all probability use this method when initially determining off-site doses

- a. RE 12444C Unit 1 Plant Vent
- b. RE 12444C Unit 2 Plant Vent
- c. RE 12839C Unit 1 & 2 Turbine Building
- d. RE 005/006 Containment Bldg Upper Level Area Rad-High Range
- e. RE 13119 S/G (4) Main Steam Line
- f. RE 13120 S/G (1) Main Sceam Line
- g. RE 13121 S/G (2) Main Steam Line
- h. RE 13122 S/G (3) Main Steam Line
- The radiation monitor readings are then used as input data with worksheets in the procedure

			LO-LP-40101-11
111.	LESSON (	DUTLINE	NOTES
	8.	The worksheet then yield the following information.	Objective 43
		a. Todine dose at site boundary	
		b. Emergency Classification	
		c. Protective action recommendations	
	OI C	crol Room Instrumentation Output for Assessment Core Damage/Core Damage Assessment Ocedure 91502-C)	
	1.	Provides a method of classify and estimating the extent of core damage	91503-C
	2.	coordinated by chemistry personnel	
	3.	Chemistry makes all damage estimates using information that may be provided by other groups	Objective 47
	4.	Operations department	
		a. Collect and record information obtained from Control Room instrumentation needed in assessment of core damage	Objective 46 91503-c
		b. RVLIS reading & recording	Objective 48
		1) Determines if core was uncovered	Section 5.1.1
		2) If so, estimate length of time in minutes	
		c. Core exit thermocouple temperatures	Section 5.2.2
		1) All temperatures exceeding 1300°F	
		2) Corresponding core location and TC ID number on Data Sheet	
		d. Containment high range monitor and containment hydrogen monitor readings	Section 5.3.1

2) BAST

e. Determine volume of all RCS additions during ascident (Data Sheet 2)

- 3) Accumulators (26,900 gallons)
- Data sheets transmitted to Chemistry department to coordinate with core damage assessment activities
  - a. If core was never uncovered and no core exit RC temperatures corresponding to saturation temp at RCS pressure were recorded, then no generalized core damage is probable
  - b. The following Tables/Data sheets from 91502 are used by Chemistry to interpret data from CR and assess damage
    - 1) TC temperature 1300°F; Table 1

ment HR monitor and H, Conc.

2) Containment HR monitor and H<sub>2</sub> Conc. (Data Sheet 1, Sheet 2)

 Chemistry also requests post accident samples

P. EMERGENCY CLASS DETERMINATION SCENARIO

1. Scenario #1

- a. A mechanic working inside a highly contaminated area suffers a deep laceration on his forearm that will require stitches and specialized medical attention. Surveys of the wound indicate internal contamination. The mechanic is sent to the hospital
- b. Notification of Unusual Event
- 2. Scenario #2
  - a. Radical anti-nuclear terrorist aided by disgruntled plant employees gain access to the protected area and the control room with intruders controlling the actions of the plant operators
  - b. Answer: General Emergency loss of physical control of plant
- 3. Scenario #3

LO-TP-40101-021 Handout #1, Page 19

Objective 13

LO-TP-40101-022

LO-TP-40101-023

- a. Following a secondary side steam break (nutside of containment) that occurred with the RCS at 450 r and 1950 page, all cold leg temperatures drop to 240 r in less than one hour
- LO-TP-40101-024

- b. Alert classification
- 4. Scenario #4
  - a. Following a moderate LOCA, pressurizer level cannot be maintained via the normal charging flowpath, even with letdown isolated

LO-TP-40101-025

b. Site Area Emergency

### End: IEN 84.042

## III. SUMMARY

- F. Review the Objectives
  - 1. NAME THE KEY INDIVIDUAL RESPONSIBLE FOR THE IMPLEMENTATION OF THE EPIPS

Emergency Director (ED)

2. STATE THE FOUR EMERGENCY CLASSIFICATIONS IN ORDER OF SEVERITY FROM LEAST TO WORST

Notification of Unusual Event (NUE) Alert Site Area Emergency General Emergency

- 3. GIVEN A LIST OF KEY EPIP , DESCRIBE THE PURPOSE AND USE OF EACH
  - A. Duties of the Emergency Director (91102-C)
     Provides instructions to the ED in
    fulfilling his responsibility for overall
    coordination and direction of the Emergency
    Response Organization
  - b. Emergency Classification (91001-C) - Provides instructions for the classification of off-normal events into one of four emergency classification levels. It also provides initial implementing instructions to the ED
  - Emergency Notification (91002-C)
     Provides instructions to complete initial

and follow up notification to GPC on-site and off-site emergency response personnel and to Georgia, South Carolina, local counties, and NRC officials

- d. Site Evacuation (91403-C) - Provides instructions for the early dismissal or evacuation of non-essential personnel from the plant site in the event of an emergency
- e. Assembly and Accountability (91401-C)
   Provides instructions for the assembly
  of non-essential personnel to their
  designated assembly area and the accounting
  of personnel to identify those who may be
  missing
- f. Protective Action Guidelines (91305-C) Provides instructions for the on-site and off-site protective actions alternatives to ensure plant and off-site personnel safety in the event of an emergency
- g. Recovery (91501-C)
   Provides instructions for determining when the emergency condition will be terminated and the recovery phase initiated. It also describes the process used to notify and activate the recovery organization for immediate and long term recovery operations
- h. Core Damage Assessment (91502-C)
   Provides a method to classify and
  estimate the extent of core damage through
  core fission product release measurements,
  reactor vessel level indications, and core
  exit thermocouple temperatures along with
  other auxiliary readings
- 4. DESCRIBE THE RESPONSIBILITY OF THE TSC, OSC, EOF, RECOVERY ORGANIZATION, AND HP AND CHEMISTRY GROUPS, AND STATE WHO DEPLOYS THEM

TSC - activated by ED

Provides in-plant radiological assessment and protective actions recommendations

Provides engineering analysis for Control Room support

Provides liaison between TSC and CR

Coordination of emergency maintenance

Directs post accident and plant chemistry sampling and analysis. Evaluates chemistry data and provides core damage assessment

Plans and implements logistical support

Provides access control and accountability in TSC

OSC - activated by ED

The functions of the OSC are:

- a. Serve as the assembly and staging area for personnel pooled for emergency response
- b. Raspond to requests from the ED and the TSC concerning deployment of RET's
- Management of emergency equipment and supplies
- d. Coordination of movement of personnel on-site

EOF - activated by ED

The functions of the EOF group are:

- Direction and control of GPC off-site emergency response
- b. Assistance to the ED and supervision of EOF
- Communication of radiological information to state and local emergency response agencies
- d. Support of initial activities associated with planning for re-entry or recovery operation
- G. Access control for EOF
- f. Processing of personnel who require authorization to enter site
- g. Requesting assistance from local law

authorities

- h. Performance of off-site dose calculation
- i. Direction of Field Monitoring Teams
- j. Coordination with state and federal groups performing radiological assessment
- Development of protective action recommendations
- Assumption of logistics support functions from the TSC
- m. Providing press release when authorized by the ED

Recovery Organization - Deploy by ED

Responsible for:

- a. Assuming assigned positions and ensuring that support staff are available and properly briefed
- Ensuring that relief personnel are fully briefed prior to relinquishing their reaponsibilities

Health Physics Group - Daployed by ED

The Health Physics Group provides the following key functions during an emergency:

- a. Performs off-site dose calculations
- b. Discusses on-site and inplant radiological conditions
- Provides on-site and off-site radiation protection measures

Chemistry Group - Deployed by ED

The Chemistry Group provides the following key function during an emergency

- a. Assess core damage from accident
- b. Evaluates in-plant chemistry analysis

## c. Operates PASS

5. LIST THE VARIOUS RADIOLOGICAL TEAMS, DESCRIBE THEIR FUNCTION, AND STATE WHO DEPLOYS THEM

Function of RET's - Deployed by ED

- a. Search and Rescue Team performs emergency search and rescue of individuals that are unaccounted for or disabled
- b. First Aid Team performs first aid for radiological and non-radiological injuries and personnel decontamination as required
- c. Damage Assessment/Control Team a selected group of personnel who perform damage assessment and control activities as needed
- d. Repair and Modification Team a solected group of personnel who perform repair work and modify systems, equipment, etc., as needed
- e. In-Plant Monitoring Team performs inplant sampling and surveys. The Health Physics Supervisor uses this information when providing Protective Action recommendations to the ED
- f. Back-up Fire Brigade back-up support to the normal shift fire brigade
- 6. STATE WHO FILLS THE INITIAL ED POSITION WHEN THE PRIMARY IN NOT ON-SITE

osus

7. STATE WHO THE PPIMARIES AND FIRST ALTERNATES ARE FOR THE ED POSITION

Primaries:

Vice President, Nuclear or General Manager, Nuclear Plant

First Alternate:

Assistant General Manager, Plant Operations

8. STATE FROM MEMORY ED DUTIES THAT CAN NOT BE DELEGATED (SRO ONLY)

- a. Classifying and declaring the emergency, including upgrading, downgrading, or termination
- Recommending protective actions to off-site authorities and control of messages
- c. Deciding to evacuate non-essential personnel from the site at the Alert classification level
- d. Authorizing personnel radiation evecures in excess of 10CFR20 limits, if I deceay
- e. Deciding to request assistance from federa's support groups
- f. Deciding to notify off-site authorities responsible for emergency measures
- 9. STATE WHO HAS AUTHORITY TO .PPI TE MODIFICATIONS TO EPIPS DURING AN EXERGENCY SITUATION

Emergency Director (ES)

10. LIST THE THREE FISSION PRODUCT BARRIERS THAT ARE FART OF THE CRITERIA FOR CLASSIFYING AN EMERGENCY

Fuel/cladding RCS Containment

11. DESCRIBE HOW THE STATUS OF FISSION PRODUCT BARRIER INTEGRITY IS OBTAINED

First determined by the operator by monitoring the CSF's using the SPDS/ERF computer or 19200-2

12. LIST AN EXAMPLE FOR THE FOLLOWING KEY FACTORS
THAT ARE CONSIDERED WHEN CLASSIFYING AN
EMERGENCY:

#### NOE:

- a. PLANT ELECTRIC POWER

  Loss of on-site or off-site power
- b. TECHNICAL SPECIFICATIONS

Loss of Engineered Safety Feature, PERMS monitor or Effluent Air Flow Instrequiring shutdown by Technical Specifications

Exceeding RCS leak rates per Tech Specs

Exceeding RCS activity per Tech Specs

Any Tech Spec safety limit violation

#### C. RADIOACTIVITY

Transportation off-site of a contaminated injured victim

Gross failed fuel monitor exceeds E+4 cpm above normal

Radioactive effluent Tech Specs

# d. PLANT SYSTEMS

Rapid depressurization of the secondary system

Turbine rotating component failure causing rapid plant shutdown

ECCS discharge to reactor vessel

Reactor protection actuation without subcriticality

Failure of a safety or relief valve to reclose in a safety related system

#### e. NATURAL PHENOMENON

Any earthquake felt or defected on-site

Tornado or hurricane

Fire in the plant lasting longer than 10 minutes

## f. MAN-MADE HAZARDS

Aircraft crash on-site

Train derailment on-site

On-site explosion

On-site toxic gas release

q. SECURITY

Unusual aircraft activity overhead Security threat or attempted sabotage

h. FISSION PRODUCT BARRIER

Not considered for this classification

1. SHUTDOWN SYSTEMS

Not considered for this classification

Alert, Site Area, and General: See 91001-C
Page 7 of 12

 GIVEN AN EMERGENCY SCENARIO, STATE THE EMERGENCY CLASSIFICATION

Refer to LP Section II.U. for examples of scenarios

14. DESCRIBE THE PROCEDURE FOR DOWNGRADING AN EMERGENCY

An emergancy is downgraded as follows:

- a. The ED determines that conditions are favorable for downgrading an emergency
- b. Notification will be made to state, local and federal officials notifying them that the emergency has been downgraded
- c. Complete all actions that are necessary for that emergency classification
- 15. STATE THE INDIVIDUAL RESPONSIBLE FOR MAKING EMERGENCY NOTIFICATIONS

Emergency Director (ED)

16. LIST THE STATE AND FEDERAL AUTHORITIES THAT ARE NOTIFIED IN AN EMERGENCY

Georgia Burke County Aiken County Barnwell County Allendale County NRC SRS

- 17. STATE THE ALLOTTED TIME TO CONTACT THE:
  - a. NRC

As soon as possible after notifications to state and county agencies

and

Within 60 minutes

b. STATE AND LOCAL AUTHORITIES

Within 15 minutes

18. DESCRIBE WHEN FOLLOW-UP MESSAGES ARE REQUIRED FOR STATE AND LOCAL OFFICIALS (SRO ONLY)

Whenever there is a significant change in plant conditions or at least every 30 minutes

19. DESCRIBE WHEN FOLLOW-UP MESSAGES ARE REQUIRED FOR THE NRC (SRO ONLY)

Whenever there is any further degradation, any change in one class to another, or termination of the emergency

20. DESCRIBE THE COMMUNICATION SYSTEM FOR NOTIFYING STATE AND LOCAL OFFICIALS

Emergency Notification Network (ENN) black phones, energized by lifting the receiver. To speak, the button on the hand set must be depressed

21. DESCRIBE THE COMMUNICATION SYSTEM FOR NOTIFYING FEDERAL (NRC) OFFICIALS

Emergency Notification System (ENS). Black phone with red dial face. It is activated by lifting the receiver

22. LIST THE BACKUP COMMUNICATIONS FOR NOTIFYING STATE AND LOCAL AUTHORITIES

#### South Carolina:

- 1) SC Backup ENN (two digit phone)
- 2) Commercial telephones
- 3) SRP Radio in TSC

#### Georgia:

- 1) Commercial telephone
- 2) Burke County Radio in TSC
- 23. LIST THE BACKUP COMMUNICATIONS FOR NOTIFYING FEDERAL OFFICIALS

The commercial phone system

24. STATE THE CIRCUMSTANCES REQUIRING SIZE EVACUATION (SRC ONLY)

Whenever a Site Area or General Emergency is declared

22

Whenever the E.D. has determined that conditions warrant it

25. STATE THE INDIVIDUAL RESPONSIBLE FOR CONDUCTING THE EVACUATION

The Security Coordinator

26. STATE THE TWO ASSEMBLY AREAS FOR VEGP NON-ESSENTIAL PERSONNEL

VEGP recreation facility - primary

Plant Wilson - backup

27. STATE THE CIRCUMSTANCES REQUIRING ASSEMBLY OF VEGP NON-ESSENTIAL PERSONNEL

Emergency Level Alert or higher

28. STATE THE GROUP RESPONSIBLE FOR PERSONNEL ACCOUNTABILITY

Security Coordinator

29. STATE THE POSITION RESPONSIBLE FOR REQUESTING A SEARCH AND RESCUE TEAM

Emergency Director

30. STATE THE POSITION RESPONSIBLE FOR FORMING THE SEA: CH AND RESCUE TEAM

OSC Manager

or

If OSC is not activated the onahift Health Physics Foreman

31. STATE THE POSITION RESPONSIBLE FOR DISPATCHING THE SEARCH AND RESCUE TEAM

OSC Manager or HP Foreman if OSC not activated

32. DESCRIBE THE INDIVIDUALS THAT EACH SEARCH AND RESCUE TEAM MUST CONSIST OF

Two members familiar with the plant and first aid training

33. STATE THE SOLE AUTHORITY RESPONSIBLE FOR AUTHORIZING DOSES GREATER THAN 10CTR20 EXPOSURE LIMITS

Emergency Director

- 34. STATE THE EMERGENCY WHOLE BODY LIMITS FOR THE FOLLOWING (SRO ONLY)
  - a. SAMPLING UNDER ACCIDENT CONDITIONS

5 Rem

b. CORRECTIVE OR PROTECTIVE ACTIONS

25 Rem

c. LIFE SAVING ACTIONS

75 Rem

35. STATE WHAT GROUP OF PEOPLE SHOULD BE FIRST CONSIDERED FOR EMERGENCY EXPOSURE, AND WHAT GROUP SHOULD NOT BE ALLOWED TO RECEIVE AN EMERGENCY EXPOSURE (SRO ONLY)

Volunteers above the age of 45

Females of child-bearing age and capability

- 36. STATE THE FOUR POSSIBLE ONSITE PROTECTIVE ACTION MEASURES AVAILABLE TO THE ED
  - a. Early dismissal
  - b. Plant evacuation
  - c. Distributing KI tablets
  - Use of additional dosimetry, respiratory protection, and protective clothing
- 37. STATE THE TWO POSSIBLE OFFSITE PROTECTIVE ACTION MEASURES AVAILABLE TO THE ED
  - a. Seek shelter
  - b. Evacuation
- 38. STATE WHO IS RESPONSIBLE FOR THE TERMINATION OF AN EMERGENCY AND NOTIFICATION TO FEDERAL, STATE, AND LOCAL AUTHORITIES

Emergency Director

- 39. DESCRIBE THE FIVE PREREQUISITES AS LISTED IN THE PROCEDURE THAT MUST BE FACTORED INTO THE DECISION TO TERMINATE AN EMERGENCY
  - Plant radiation levels are stable or decreasing with time
  - b. The affected reactor is in stable condition and can be maintained indefinitely in a stable condition
  - c. Fire or other similar emergency conditions no longer constitute a hazard to safety-related systems, equipment, or personnel
  - d. Radioactive releases to the environment have ceased or been controlled to within permissable license limits
  - e. Discussions with GPC management, VEGP Emergency Response Organization, and offsite authorities do not result in identification of any valid reason for not

terminating the emergency

40. STATE THE POSITION RESPONSIBLE FOR RECOVERY OPERATIONS AND THE PRIMARY FOR THAT POSITION

Recovery Manager

Primary is General Manager, Nuclear Plant

- 41. STATE WHO IS RESPONSIBLE FOR PERFORMING OFF-SITE DOSE CALCULATIONS
  - a. INITIALLY

OSOS or designee

- b. AFTER THE TSC IS ACTIVATED Health Physics Supervisor
- O. AFTER EOF IS ACTIVATED

  Dose Assessment Hanager
- 42. DESCRIBE THE INPUT AND WHERE IT IS OBTAINED IN THE MANUAL BACKUP CALCULATION

Radiation monitor readings from plant vent stack U1/2, Turbine Building U1/2, Containment Building, or main steam lines

43. DESCRIBE WHAT OUTPUT IS OBTAINED FROM THE MANUAL BACKUP CALCULATION

Iodine dose at site boundary Emergency classification Protective action recommendations

44. DESCRIBE THE INPUT USED FOR COMPUTER CALCULATION OF OFFSITE DOSES

Wind speed and direction Plant radiological effluent data Direct measurement of effluent path with a postable instrument

45. DESCRIBE THE OUTPUT THAT IS OBTAINED WHEN USING THE COMPUTER TO OBTAIN OFFSITE DOSE CALCULATION

Plume arrival time Direction (X/Q) Whole body doses Infant thyroid doses Doses at preset downwind distances

46. STATE THE GROUP RESPONSIBLE FOR COLLECTION OF DATA FOR CORE DAMAGE ASSESSMENT

Operations Department

47. STATE THE GROUP RESPONSIBLE FOR ASSESSING CORE DAMAGE

Chemistry Department

48. DESCRIES THE DATA AND THE INSTRUMENTATION THAT IS USED IN THE ASSESSMENT OF CORE DAMAGE

RVLIS reading and recordings
Core exit thermocouple temperatures
Contemnment high range monitor and containment
hydrogen monitor readings
Volume of all RCS additions during accident

49. STATE WHO IS RESPONSIBLE FOR ACTIVATION OF THE TSC, OSC, AND EOF

Emergency Director (ED)

50. STATE THE PRIMARY AND BACKUP LOCATION AND THE CRITERIA USED TO DETERMINE UPPER LIMIT OF HABITABILITY OF THE PRIMARY TSC, OSC, AND EOF

Whole body dose 100 mr/hr in TSC, OSC, EOF Iodine activity 2.7 E-07 mCi/cc in TSC, OSC, EOF

TSC - Primary TSC Backup Control Room

OSC - Primary OSC Backup TSC

EOF - Primary EOF Backup GPC District Office Waynesboro, Georgia Following completion of this lesson, the student will possess those knowledges systematically identified for the performance of the EPIP OVERVIEW tasks.

# II LIST OF OBJECTIVES

- Name the key individual responsible for the implementation of the EPIPs.
- State the four emergency classifications in order of severity from least to worst.
- 3. Given a list of key EPIPs, describe the purpose and use of each.
- Describe the restonsibilities of the TSC, OSC, EOF, Recovery Organization, and HP and Chemistry groups during a declared emergency.
- List the various radiological teams, describe their function, and state who deploys them (91102-C). (SRO only)
- State who fills the initial ED position when the primary is not on-site.
- 7. State who the primaries and alternates are for the ED position.
- 8. State from memory ED duties that cannot be delegated.
- 9. State who has authority to approve modifications to EPIPs during an emergency situation (91001-C).
- List the three fission product barriers that are part of the criteria for classifying an emergency.
- 11. Describe how the status of fission product barrier integrity is obtained.

# OBJECTIVES FOR LO-LP-40101 CONTINUED.

12. List and the the following key factors that are considered when class than a mergancy:

a. Plant electric power

- b. Technical Specifications
- c. Radioactivity
- d. Plant Systems e. Natural Phenomena f. Mon-Made Hazards

g. Security h. Fission Product Barrier

1. Shutdown Systems

- 1) Loss of systems necessary to put plant in hot shutdown 2) Loss of systems necessary to put plant in cold shutdown
- 13. Given an emergency scenario, state the emergency classification.
- 14. Describe the procedure for downgrading an emergency (91002-U). (SEO ONLY)
- 15. State the individual responsible for making emergency notifications.
- 16. List the state and federal authorities that are notified in an emergency.
- 17. State the allotted time to contact the (SRO ONLY): a. NRC b. State and local authorities
- 18. Describe when follow-up ressages are required for state and local officials. (SRO ONLY)
- 19. Describe when follow-up messages are required for the NRC (91204-C). (SRO ONLY)
- 20. Describe the communication system for notifying state and local officials.
- 21. Describe the communication system for notifying federal (NRC) officials.
- 22. List the backup communications for notifying state and local authorities. (SRO ONLY)

# OBJECTIVES FOR LO-LP-40101 CONTINUED.

- 23. List the backup communications for notifying federal officials (91403-C). (SRO ONLY)
- 24. State the circumstances requiring site evacuation.
- 25. State the individual responsible for conducting the evacuation (91401-0).
- 26. State the two assembly areas for VEGP non-essential personnel.
- State the circumstances requiring assembly of VEGP non-essential personnel.
- 28. State the group responsible for personnel accountability (91402-C). (SRO only)
- 29. State the position responsible for requesting a search and rescue team.
- 30. State the position responsible for forming the search and rescue team.
- 31. State the position responsible for dispatching the search and rescue team.
- 32. Describe the individuals that each search and rescue team must consist of (91301-C).
- 33. State the sole authority responsible for authorizing doses greater than 10CFR20 exposure limits.
- 34. State the emergency whole body limits for the following (SRO ONLY):
  - a. Sampling under accident conditions
  - b. Corrective or protective actions
  - c. Life saving autions
- 35. State what group of people should be first considered for emergency exposure, and what group should not be allowed to receive an emergency exposure (91305-C). (SEO only)

# OBJECTIVES FOR 10-LP-40101 CONTINUED. 35. Summurize the evaluation and implementation of Protective Action Guidelines as specified in the EFIF's. 37. State the following two possible off-site protective action measures available to the ED (91301-C): (SRO only) a. Seek shelter b. Evacuation 38. State who is responsible for the termination of an emergency and notification to federal, state, and local authorities. 39. Describe the five prerequisites as listed in the procedure that must be factored into the decision to terminate an smergency. 40. State the position responsible for recovery operations and the primary for that position (91304-C). 41. State who is responsible for performing off-site dose calculations a. Initially b. After the TSC is activated c. After the EOF is activated 42. Describe the input and where it is obtained in the manual backup calculation. 43. Describe what output is obtained from the manual backup calculation. 44. Describe the input used for computer calculation of off-site doses.

43. Describe the output that is obtained when using the computer to

46. State the group responsible for collection of data for core damage

obtain off-site dose calculation (91503-C).

47. State the group responsible for assessing core damage.

48. Describe the data and the instrumentation that is used in the assessment of core damage (91202-C, 91203-C). (SRO only)

REPRESENTE.

# ORJECTIVES FOR LO-LF-40101 CONTINUED.

- 49. State who is responsible for activation of the TSC, OSC, and EOF.
- 50. State the primary and backup location and the criteria used to determine upper limit of habitability of the primary TSC, OSC, and EOF.