Dockhold	NUCLEAR OPERATIONS	Procedure No. 12007-C
3/8/90	Unit_COMMON Georgia Power	Revision No. 14 Page No.
UNIT NO.		1 of 24
	DATE	
	REFUELING ENTRY	
	(MODE 5 TO MODE 6)	
1.0	PURPOSE FOR INFO	
	This procedure provides instructions for unit from a cold shutdown (Mode 5) with Re temperature between 80 and 130 degrees, to condition (Mode 6), and initiating core al	aking the actor Coolant refueling terations
2.0	PRECAUTIONS AND LIMITATIONS	
2.1	PRECAUTIONS	
2.1.1	If this procedure is terminated prior to o the Unit Shift Supervisor (USS) should not for the termination in the comments section	ompletion, e the reason
2.1.2	Notify Health Physics prior to performing evolutions which may significantly alter r levels.	operations adiation
2.1.3	Notify Chemistry prior to installing or re Containment Equipment Hatch that containme ventilation flow will be changed during th	moving the nt
2.1.4	During periods of operation with the React System (RCS) level below the Reactor Vesse elevation (194 feet elevation), ongoing wo should be closely scrutinized and any work limited that has the potential for reducin capability.	or Coolant l Flange rk activities activity g RHRS
2.1.5	Inadvertent Containment Ventilation Isolat occur during the movement of the Reactor V from the cavity to the head stand. Ensure Physics initiates compensatory actions to inadvertent actuations	ion (CVI) may essel Head Health prevent

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2.2	LIMITATION	S		•
2.2.1	The RCS pr psig and 3 Removal (R	essure and tem 50 degrees whe HR) Syst m.	perature sh n open to t	nall not exceed 425 the Residual Heat
2.2.2	In Mode 5, equal to t 3.1.1.2, F	shutdown marg he limit speci igure 3.1-2.	in shall be fied in Teo	e greater than or chnical Specificati
2.2.3	While in M with the R fully tens maintained shall be m whichever 1208-U4-17 shall be c stops), ex opened for provided t a setpoint background	ode 6 (wheneve eactor Vessel ioned or with at 0.95 or le aintained grea is more restri 5, 1208-U4-177 losed and secu cept 1208-U4-1 short periods he Hi Flux at of less than . (Technical	r fuel is in Head Closur the head read ss, or the ter than or ctive. Add , 1208-U4-1 red in posi 76 and 1208 of time for Shutdown Al or equal to Specificati	in the Reactor Vess re Bolts less than moved) Keff shall boron concentration requal to 2000 ppm litionally, valves 83 and 1208-U4-176 tion (by mechanica 1-U4-177 may be or chemistry contro arm is operable wi 2.30 times on 3.9.1)
2.2.4	When in Mo Train shal	de 5, with loop 1 be operable	ps filled, and in oper	at least one RHR ation, and either:
	a. One a	dditional RHR	train shall	be operable, or
	b. The s Steam range	econdary side Generators sha level. (Tech	water level all be grea nical Speci	of at least two ter than 17% of wid fication 3.4.1.4.1
2.2.5	While in M trains sha shall be in 1208-U4-17 shall be c stops), ex opened for provided th a setpoint background	ode 5 with the 11 be operable n operation. 1 5, 1208-U4-176 losed and secur cept 1208-U4-1 short periods he Hi Flux at 1 of less than 6 . (Technical 1	RCS loops and at lea Reactor Mak , 1208-U4-1 red in posi 76 and 1208 of time fo Shutdown Al or equal to Specificati	not filled, two RHI st one RHR train eup Water Valves 77, and 1208-U4-18 tion (by mechanica -U4-177 may be r chemistry contro arm is operable with 2.30 times on 3.4.1.4.2)
2.2.6	When in Mo equal to 2 least one 1 (Technical	de 6, with the 3 feet above th RHR train shal Specification	water leve he Reactor l be operab 3.9.8.1)	l greater than or Vessel Flange, at le and in operation
2.2.7	When in Mo above the 1 be operable (Technical	de 6, with the Reactor Vessel e and at least Specification	water leve Flange, tw one RHR tr 3.9.8.2)	el less than 23 fee o RHR trains shall ain in operation.

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2.2.8	While in Mc	des 4, 5, and 5 wit	h the Reactor Vessel Hea
	on, at leas	t one of the follow	ing Cold Overpressure
	Protection	Systems (COPS) shal	1 be operable:
	a. Two Po settin establ 3,4-4,	ower Operated Relief gs which do not exc ished in Technical or	Valves (PORV) with lift eed the limits Specification Figure
	b. Two RH of 450	IR Suction Relief Va psig ±3%, or	lves each with a setpoin
	c. The RC	S depressurized wit	h an RCS vent capable of
	reliev	ing at least 670 gp	m water flow at 470 psig
	(Techn	ical Specification	3.4.9.3)
2.2.9	While in Mo	des 5 and 6, at lea	st one of the following
	Boron Injec	tion Flow Paths sha	11 be operable.
	a. A flow	path from the Bori	c Acid Storage Tank via
	Boric	Acid Transfer Pump	and a Charging Pump to
	the Re	actor Coolant System	n if the Boric Acid
	Storag	e Tank is operable,	or
	b. The fl	ow path from the Re	fueling Water Storage
	Tank (	RWST) via a Chargin	g Pump to the Reactor
	Coolan	t System if the Ref	deling Water Storage Tan
	is ope	rable. (Technical S	Specification 3.1.2.1)
2.2.10	The tempera coolant in 70 degrees Steam Gener Specificati	ture of both the pritte the Steam Generators when the pressure of ator is greater than on 3.7.2)	imary and secondary s shall be greater than f either coolant in the h 200 psig. (Technical
2.2.11	While in Mo Nuclear Ins NR-45 and t alarm opera	de 5 at least one cl trumentation should he CONTROL ROOM HI 1 ble.	nannel of Source Range be selected to Recorder FLUX LEVEL AT SHUTDOWN
2.2.12	While in Mo shall be op the Control Containment 3.9.2)	de 6 both Source Ram erable with continue Room and one with a and Control Room.	nge Neutron Flux Monitor ous visual indication in audible indication in th (Technical Specificatio
2.2.13	The reactor	shall have been sub	ocritical for at least 1
	hours prior	to moving irradiate	ad fuel in the Reactor
	Pressure Ve	ssel. (Technical Sp	Decification 3.9.3)
2.2.14	During Core	Alterations, direct	communications shall b
	maintained	between the Control	Room and personnel at

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2.2.15	While in M Reactor Ve the RWST w 99,404 gal boron conc	odes 5 and 6, with the 1 ssel Flange elevation (1 ill be operable with a r long (9% of instrument a entration between 2400 a	RCS level below 194 feet elevation), minimum volume of span) of water at a and 2600 ppm.
3.0	INITIAL CO	NDITIONS	
3.1	The RHR Sy 3000 gpm an	stem is in operation at nd RHR letdown is in ser	a minimum flow of rvice.
3.2	Sufficient ordered to	Carbon Dioxide and Nits support plant operation	rogen is on hand or ns.
3.3	If required the RWST at to support	d, there is sufficient of t a minimum boron concer refueling operations.	volume available in ntration of 2400 ppm

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<ul> <li>UNIT NO</li></ul>	NSTRUCTIO DDE 5 AND DDE 5 AND 2 pressu 7 feet e ontrols s Tygon time while (appr press (1)	O 6 OPERATIONS NOTE Asterisk (*) ste INITIALS spaces steps that gener additional docum aving with the F rizer level (app levation) the for hould be in effe tube watch is r the RCS level is the RCS level is the RCS level is the RCS level is the RCS level is oximately 207 fe urizer level, Periodic compari should be made e between the Cont Temporary RCS Level 1	eps boside indicates tate ments. ACS level b proximately pilowing ect: required and being chan being chan being chan to be ow 17 tet elevations son checks every 4 hour	elow nged ion)	INITIAL
<ul> <li>A.1 MODE 5 AND 6 OPERATIONS</li> <li>4.1 MODE 5 AND 6 OPERATIONS</li> <li>Asterisk (*) steps baside INITIALS spaces indicates steps that generate additional documents.</li> <li>4.1.1 While operating with the RCS level below 177 pressurizer level (approximately 207 feet elevation) the following controls should be in effect: <ul> <li>Tygon tube watch is required any time the RCS level is below 177 (approximately 207 feet elevation) pressurizer level.</li> <li>Periodic comparison checks should be made every 4 hours between the Control Room Temporary RCS Level Monitors aud the Tygon tube,</li> <li>Two out of three Level Monitors should agree within 7 percent of scale with the Tygon tube,</li> <li>Two out of three Level Monitors must agree before draining RCS below the top of the hot leg (188 feet 3 inches),</li> </ul> </li> <li>4) If neither Control Room RCS Level Monitor is available, then a continuous Tygon tube watch should be established while RCS level is below 172 preservices level</li> </ul>	VSTRUCTIO DDE 5 AND DDE 5 AND 2 pressu 7 feet e ontrols s Tygon time while (appr press (1)	ONS 0 6 OPERATIONS NOTE Asterisk (*) ste INITIALS spaces steps that gener additional docum ating with the H rizer level (app levation) the for hould be in effe tube watch is r the RCS level is the RCS level is the RCS level is oximately 207 fe urizer level, Periodic compari should be made e between the Cont Temporary RCS Le	eps boside indicates rate ments. CS level b proximately plowing act: required an being char s below 17 ret elevations son checks every 4 hour	elow nged Z on)	INITIAL
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<ul> <li>NOTE</li> <li>NOTE</li> <li>Asterisk (*) steps boside INITIALS spaces indicates steps that generate additional documents.</li> <li>4.1.1 While operating with the RCS level below 17% pressurizer level (approximately 207 feet elevation) the following controls should be in effect: <ul> <li>Tygon tube watch is required any time the RCS level is being changed while the RCS level is being changed while the RCS level is being changed while the RCS level is below 17% (approximately 207 feet elevation) pressurizer level.</li> <li>Periodic comparison checks should be made every 4 hours between the Control Room Temporary RCS Level Monitors aud the Tygon tube,</li> <li>The Control Room Monitors should agree vithin 7 percent of scale with the Tygon tube,</li> <li>Two out of three Level Monitors must agree before draining RCS below the top of the hot leg (188 feet 3 inches),</li> <li>If neither Control Room RCS Level Monitor is available, then a continuous Tygon tube watch should be established while RCS level is below 17% preservicer level</li> </ul> </li> </ul>	nile oper 72 pressu 77 feet e ontrols s Tygon time while (appr press (1)	NOTE Asterisk (*) ste INITIALS spaces steps that gener additional docum aving with the F rizer level (app levation) the fo hould be in effe tube watch is r the RCS level is the RCS level is the RCS level is oximately 207 fe urizer level, Periodic compari should be made e between the Cont Temporary RCS Le	eps boside indicates rate ments. RCS level b proximately plowing act: required and being chails below 17 ret elevations son checks every 4 hour	elow nged I on)	
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<ul> <li>Asterisk (*) Steps beside</li> <li>INITIALS spaces indicates</li> <li>steps that generate</li> <li>additional documents.</li> <li>4.1.1 While operating with the RCS level below</li> <li>17% pressurizer level (approximately</li> <li>207 feet elevation) the foilowing</li> <li>controls should be in effect: <ul> <li>a. Tygon tube watch is required any</li> <li>time the RCS level is being changed</li> <li>while the RCS level is below 17%</li> </ul> </li> <li>(1) Periodic comparison checks <ul> <li>should be made every 4 hours</li> <li>between the Control Room Monitors</li> <li>aud the Tygon tube,</li> </ul> </li> <li>(2) The Control Room Monitors <ul> <li>should agree vithin 7 percent</li> <li>of scale with the Tygon tube,</li> </ul> </li> <li>(3) Two out of three Level Monitors <ul> <li>must agree before draining RCS</li> <li>below the top of the hot leg</li> <li>(188 feet 3 inches),</li> </ul> </li> <li>(4) If neither Control Room RCS <ul> <li>Level Monitor is available,</li> <li>then a continuous Tygon tube</li> <li>watch should be established</li> <li>while RCS level is below 17%</li> <li>presenviser level</li> </ul></li></ul>	nile oper 7% pressu 7% feet e ontrols s Tygon time while (appr press (1)	Asterisk (*) ste INITIALS spaces steps that gener additional docum aving with the F rizer level (app levation) the for hould be in effe tube watch is r the RCS level is the RCS level is oximately 207 fe urizer level, Periodic compari should be made e between the Cont Temporary RCS Le	eps boside indicates rate ments. RCS level b proximately plowing ect: required any being char s being char s below 17 ret elevations son checks every 4 hour	y nged I on)	
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hrepourteer react.	43	If neither Contr Level Monitor is then a continuou watch should be while RCS level pressurizer leve	ol Room RC available is Tygon tul established is below 1 1.	S be d 7%	
		(3)	<ul> <li>(3) Two out of three must agree befor below the top of (188 feet 3 inch (188 feet</li></ul>	<ul> <li>(3) Two out of three Level Mon must agree before draining below the top of the hot 1 (188 feet 3 inches),</li> <li>(4) If neither Control Room RC Level Monitor is available then a continuous Tygon tu watch should be establishe while RCS level is below 1 pressurizer level.</li> </ul>	<ul> <li>(3) Two out of three Level Monitors must agree before draining RCS below the top of the hot leg (188 feet 3 inches),</li> <li>(4) If neither Control Room RCS Level Monitor is available, then a continuous Tygon tube watch should be established while RCS level is below 17% pressurizer level.</li> </ul>

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UNIT NO.	b.	If i less Vess the shal	t is tha el F foll l be	intended to drain on 3 feet below the F lange (191 ft. el.) owing additional cor placed in effect:	lown to Reactor then htrols	<u>int Dals</u>
		(1)	DET Con and bei cr to thr Ves	ERMINE closure statu tainment Equipment H ENSURE hatch is cap ng closed within 57 ENSURE hatch is clos reducing RCS level b ee feet below the Re sel Flange (191 ft.	as of iatch bable of minutes sed prior below eactor el.),	
		(2)	A ripen 142 Ten shou det been and thou	eview of all Contain etrations addressed 10, "Containment Bui etrations - Refuelin 11d be accomplished ermine those which h n opened by manual m an info LCO generat se identified,	iment in .lding g" to ave seans ed for	
		(3)	A mi the avai the	inimum of two incore rmocouples shall be ilable during period Reactor Head is ins	s where talled,	
		(4)	REQI dest then to a temp "Cor	JEST I&C reset the Ignated ERF incore rmocouples alarm set alarm at 10°F above perature per 00410-C nputer Software Cont	point desired rol",	
		(5)	If soperations of the second s	GG Nozzle Dams are t talled and no cold 1 ning is to be establ at path is required Reactor Vessel uppe	o be eg ished, from z plenum.	
			This by:	vent path can be s	atisfied	
			(a)	Removing a pressur manway, or	izer	
			(b)	Removing a Steam G manway on a hot le will not be dammed	enerator g that , or	
			(c)	Removing three pre	ssurizer	

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	(6)	If SG Nozzle Dams are to be installed and a cold leg opening is to be established, a very ath is required from the Reactor Vessel Upper Plenum by removing an SG manway on an HL that will not be dammed.	
	(7)	If it is intended to operate at one foot above mid-nozzle level, the preferred RHR configuration is one train operating with a flow of 3000 gpm,	
	(8)	While operating with SG Nozzle Dams installed, ENSURE one Safety Injection Pump is capable of being racked in and operated in the hot leg injection mode if needed,	
	(9)	While level is in the region of the hot legs, TREND RHR Pump parameters on ERF for early detection of possible RHR Pump degradation due to vortexing,	
	(10)	Minimum RCS level is one foot above mid-nozzle (188 feet 0 inches elevation) except for Steam Generator burping during initial drain down. For effective SG tube draining, RCS level should be lowered to 187 feet 6 inches. Upon completion of SG burping, RAISE RCS level to 188 feet - 0 inches and MAINTAIN at this level thereafter.	
	(11)	A minimum of 4 Containment Cooling Units will be operable and capable of being started if required while RCS level is below 191 feet elevation.	

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<ul> <li>UNIT NO</li></ul>	<ul> <li>UNIT NO</li></ul>
<ul> <li>UNIT NO</li></ul>	<ul> <li>UNIT NO</li></ul>
<ul> <li>4.1.2 MAINTAIN RCS temperature in range of 80 <ul> <li>to 130 degrees and a total flow of 3000 gpm by adjusting the RHR System as necessary per 13011, "Residual Heat Removal System".</li> </ul> </li> <li>4.1.3 During RCP seal package maintenance or SG primary side inspections, MAINTAIN RCS level at 188 feet - 0 inches (one foot above mid-nozzle elevation). <ul> <li>NOTE</li> <li>Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation.</li> </ul> </li> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Florer of Elevel Florer of Fl</li></ul>	<ul> <li>4.1.2 MAINTAIN RCS temperature in range of 80 <ul> <li>to 130 degrees and a total flow of 3000</li> <li>gpm by adjusting the RHR System as necessary per 13011, "Residual Heat Removal System".</li> </ul> </li> <li>4.1.3 During RCP seal package maintenance or SG primary side inspections, MAINTAIN RCS level at 188 feet - 0 inches (one foot above mid-nozzle elevation). <ul> <li>NOTE</li> <li>Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation.</li> </ul> </li> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Flange elevation).</li> <li>4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeun Control System"</li> </ul>
<ul> <li>4.1.3 During RCP seal package maintenance or SG primary side inspections, MAINTAIN RCS level at 188 feet - 0 inches (one foot above mid-nozzle elevation).</li> <li>NOTE</li> <li>Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation.</li> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet head removal Flore closed)</li> </ul>	<ul> <li>4.1.3 During RCP seal package maintenance or SG primary side inspections, MAINTAIN RCS level at 188 feet - 0 inches (one foot above mid-nozzle elevation).</li> <li>NOTE</li> <li>Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation.</li> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Flange elevation).</li> <li>4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System"</li> </ul>
NOTE Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation. 4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet	NOTE Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation. 4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Flange elevation). 4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System"
Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation. 4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet	<ul> <li>Maintain RCP seal injection in operation while RCS level is greater than 190 feet - 0 inches elevation.</li> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Flange elevation).</li> <li>4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System"</li> </ul>
4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet	<ul> <li>4.1.4 During preparation for Reactor Vessel head removal, MAINTAIN RCS level less than or equal to 192 feet (two feet below Vessel Flange elevation).</li> <li>4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System"</li> </ul>
Delow vessel Flange elevation).	4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System"
4.1.5 If the outage is for refueling, then ENSURE that the RCS has been borated to refueling concentration per 13009, "CVCS Reactor Makeup Control System".	erse neactor nakeup concrot system .

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UNIT NO.		н. Н			INITIALS
4.2	PREPARAT	IONS FOR REFU	ELING		
4.2.1	Twelve he water fro Cavity of INITIATE preparat: sample.	ours prior to om the RWST t the Reactor RWST recircu lon for RWST	transferrs o the React Vessel, lation in chemistry	or	
4.2.2	VERIFY th Canal Gat	hat the Fuel te is closed.	Pool To Tra	insfer	
4.2.3	PREPARE to level for Transfer the follo	the Refueling refueling o System check owing:	Cavity low perations a outs by per	ver Ind Fuel forming	
		N	OTE		
		This step m just prior Intent is t lead time t tube to est penetration preparation provide wat Fuel Transf	ay be defer to head lif o provide e o fill the ablish a co water seal for head l er lubricat er System c	red to t. arly transfer ntainment in ift and ion for heckouts.	
	a. PERF alig	ORM the foll	owing prefi	11	
	(1)	CLOSE and TA Isolation of unit:	AG Cavity D n the appli	rain cable	
		UNIT 1: 1	-1901-U6-26	Ο,	
					<u>I</u> V
		UNIT 2: 2	-1901-U6-26	0,	
	(2)	ENSURE Main installed th on the 12 in the Refuelin	tenance has he 2 Blind hch drain 1 hg Cavity,	Flanges ines in	- V

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	(3)	ENSURE Maintenance has completed Reactor Cavit Sealing per 93240-C, "R Vessel Assembly/Disasser Instructions",	y eactor mbly	
	(4)	ENSURE Maint usage has the Transfer Tube Blind per 93240-C, "Reactor V Assembly/Disassembly Instructions",	removed Flange essel	
	(5)	If the Transfer Canal 1 has been lowered to belo Transfer Tube elevation UNLOCK and OPEN the Tran Tube Gate Valve,	evel ow the , then nsfer	
	(6)	After the RWST has recip for a minimum of 6 hours CONTACT Chemistry to tal sample from the RWST to total suspended solids concentration is within specifications.	rculated s, ke a verify	
		If total suspended solid concentration is out of specifications, INITIATS Cleanup per 13719, "Spen Fuel Pool Cooling And Purification System".	ds E RWST nt	
		NOTE		
		If the Transfer Canal is flooded above the Transf Tube elevation with the Transfer Tube Gate Valve closed, then Step 4.2.38 may be N/A'd.	s fer o	
	b. FILL from Syst leas (app Fuel per Cool	the lower Reactor Cavity the RWST via the SFP Cod em to an elevation of at t 188 feet - 0 inches roximately 2 feet above t Transfer Tube centerline 13719, "Spent Fuel Pool ing And Purification Syst	pling the e) tem".	

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UNIT NO.	in the larger state is reasoned.	Mariana.			**	INITIALS
	c. If fi in gr Sp	the lo lled, t itiate ab samp ecifica	ower reac then NOTI daily pl bles (Tec ations Te	tor cavit FY Chemis ant vent hnical ble 4.11-	y was try to Tritium 2 Note 4).	
Person Co	ontacted	CONTRACTOR OF LOW AND A VERY		Dat	e	Time
4,2,4	PERFORM prepara applica	the fo tion va ble uni	liowing lve alig	refueling nment on	the	
	a. EN IN (O	SURE CL NER GAS SS EL -	OSED RCS KET ISO. 172 fee	RV SEAL 1	LKOFF	
	UN	IT 1:	1-1201-	04-087		And the second second
						TV
	UN	IT 2:	2-1201-	U4-087		n 7
						IV
	b. CL GA	OSE RCS SKET IS	RV SEAL	LMOFF OU EL - 172	TER feet)	
	UN	IT 1:	1-1201-	04-088		
						TV
	UN	IT 2:	2-1201-	U4-088		
						IV
	c. CL	OSE RV	LEAKOFF	ISO HV-80	32,	
						IV
	d. EN SE	AL SUPP	OSED REA	CTOR CAVI: N	ΓΥ	
	UN	IT 1:	1-1213-	U4-088		
						TV
	UN	II 2:	2-1213-	U4-088		

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UNIT NO.	and the first sector of the second second			INITIALS
4.2.5	If desired pre-heating to 80°F by HS-12470 ar	, reset the FHB HVAC g Coil Thermostat from placing local handswi nd HS-12471 to the ON	m 70°F itches position.	
4.2.6	NOT1FY Heal locked or p plugs for t Bellows in and Contair	th Physics to estable posted access on the other the Fuel Transfer Tube the Fuel Handling Bus ment Building.	ish a concrete e ilding	
4.3	MODE 6 ENTH	ΥY		
4.3.1	Prior to Ma detensionir (Mode 6 ent	intenance Department ng the first Reactor H rry), PERFORM the foll	Head Bolt lowing:	
		NOTE		
	i d t a s p t M	nto Mode 6 has slippe lelays or holds, then he Pre-refueling Chec nd reperform those ap urveillances required erformed within the s ime frames prior to e lode 6.	ed due to review cklist oplicable d to be specified entry into	
	a. INITIA comple within prior	TE Mode 6 Entry Check ting those applicable the specified time f to entry into Mode 6,	clist 1, steps Frames	195-1
	b. OBTAIN Change Survei all de survei Mode 6	from the Control Roc Binder or OBTAIN fro llance Tracking Coord ferred (not performed llance tests required entry.	om Mode om the linator l) l for	
	SCHEDU applic Mode 6	LE and COMPLETE those able test procedures entry.	prior to	

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UNIT NO.			*	INITIALS
	c. REVI ente	EW the following for impac ring Mode 6.	ct on	
	(1)	Jumper and Lifted Wire Lo	Dg,	
	(2)	Temporary Modification La	og,	
	(3)	Equipment Clearance Log,		
	(4)	LCO Book,		
	(5)	Outstanding Work Orders.		
		NOTE		
		Two RCS Core Exit Thermoor shall be maintained when level is less than 191 ft	couples RCS t. el.	
	d. COOR Super follo disa comp Vesso Inst	DINATE with the Outage Are rvisor to ensure that the owing Reactor Vessel Head ssemb y activities have be leted per 93240-C, "Reacto el Assembly/Disassembly ructions".	en or	
	(1)	Seismic Tie Rods moved,		
	(2)	Cables disconnected,		
	(3)	Head Insulation removal,		
	(4)	Head Vent piping disconne	ect,	
	(5)	RVLIS Head connection disconnected,		
	(6)	Instrument port Conoseal disassembly complete.		
4.3.2	OBTAIN On approval Mode 6.	-Shift Operations Supervis to change status from Mode	sor's e 5 to	
	0505	Signature Date	Time	
4.3.3	When notion that the H has comment Unit Control	fied by Maintenance Depart Reactor Vessel Head detens nced, LOG Mode 6 entry int rol Logbook and INITIATE M	tment sioning to the Mode 6	

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na mana i kana na datana da man	na financial e stato a stato a stato e se e secondo a stato e s		ana ana amin'ny faritr'o amin'ny faritr'o amin'ny faritr'o amin'ny faritr'o amin'ny faritr'o amin'ny faritr'o a	·····	
UNIT NO.					INITIAL
4.4	MODE 6 OPE	RATIONS			
4.4.1	In additio surveillan to initiat if applica least once Specificat	n to the sc ces, NOTIFY e Boron ana ble, the Re per 72 hou ion 4.9.1.1	heduled Mode Chemistry D lysis of the fueling Cavi rs. (Techni )	6 epartment RCS and, ty at cal	
Person Co	ntacted		Date	Ti	me
4.4.2	COMPLETE t Reactor Ve	he followin ssel Head 1	g to prepare ift:	for	
		N	OTE		
		As a precau Building Pe Technical S will be est periods of Head moveme	netrations pecification ablished dur Reactor Vess nt.	3.9.4 ing el	
	a. NOTIF the C will venti	Y Chemistry ontainment change cont lation flow	that closur Equipment Ha ainment	e of tch	
	b. NOTIF Conta Syste	Y Maintenan inment Pers m,	ce to reset onnel Lock I	the nterlock	
	c. PERFO Build Refue	RM 14210, " ing Penetra ling",	Containment tions Verifi	cation -	****
	d. ENSUR for R "Resi	E one train efueling Ca dual Heat R	of RHR is a vity fill pe emoval Syste	ligned r 13011, m",	

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UNIT NO.		CAUTTON		INITIALS
		Inadvertent Containmer Ventilation Isolation may occur during the of the Reactor Vessel from the cavity to the stand. Ensure Health initiates compensatory to prevent inadverten actuations.	nt (CVI) movement Head e head Physics y actions t	
	e. COORD Super follo compl Vesse Instr	INATE with the Outage visor to ensure that wing activities have 1 eted per 93240-C, "Rea 1 Assembly/Disassembly uctions".	Area the been actor	
	(1)	Power and Signal Cable	es removed,	
	(2)	Flux Thimbles withdraw	vn,	
	(3)	Tools removed from reactive reaction reactive.	fueling	
4.4.3	After the Outage Are filling th feet - 6 i deck) per System".	head lift, COORDINATE a Supervisor and INIT e Refueling Cavity to nches (2 feet below of 13011, "Residual Heat	with LATE 218 Derating Removal	
	During the Refueling Check Valv	process of filling th Cavity, PERFORM 14895, e Refueling Inservice	ne "ECCS Test".	
4.4.4	If the Low previously to initiat grab sampl Table 4.11	er Reactor Cavity was filled, then NOTIFY ( e daily plant vent Tri es. (Technical Specif -2 Note 4)	not Chemistry Ltium fication	
Person Co	ontacted	Date	Tim	ie
4.4.5	During the preparation MAINTAIN R	remainder of refuelin ns and core alteration efueling Cavity level	ng ns, at	

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UNIT NO.				INITIALS
4.4.6	PLACE the R System in s Fuel Pool C System".	efueling Cavity Filtratic ervice per 13719, "Spent ooling And Purification	'n	
		NOTE		
	Ri In w at t I i t b e A	emoval of the Upper nternals Assembly and ithdrawal of any RCCA ssembly in excess of nree feet from its fully nserted position within he Reactor Vessel should considered as Core lterations.		
4.4.7	Prior to mov Assembly, CO steps of Con	Ving the Upper Internals OMPLETE the applicable re Alterations Checklist	2.	
4.4.8	After the Up been set in the Refuelir stable at 21 that the fue equal to the	oper Internal Assembly ha the storage location and ng Cavity level has been 8 feet - 6 inches, VERIF el pool level is approxim e transfer pool level.	s Y ately	
	a. If the not ope then UN Tube Ga	Transfer Gate Valve was ened per Step 4.2.3a(5), NLOCK and OPEN the Transf ate Valve,	er	
	b. OPEN th Canal C	ne Fuel Pool To Transfer Gate.		
1.1.0	NOTIFY Chemi Containment	stry to reset PERMS Low Range Area Monitors		

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UNIT NO.					INITIALS
4.4.10	Prior to t loads with Pool with the Normal the Low Ne FHBI actua Actuation the follow	ransporting fu in or over the spent fuel in FHB HVAC in s gative Differe tion channels Cabinet QESF b ing:	el or othe Spent Fue the pool a ervice, UI ential Pres at the BOI by perform	er el and NBLOCK ssure P ing	
	a. VERIF is abo by ob A-ZI- hands	Y the FHB nega ove the actuat serving white 12567 and A-ZI witch A-HS-253	tive press ion setpo lights -12568 at 3C OUT,	sure int	
	b. PLACE the Ol	handswitch A- FF position.	HS-2533C 1	to	
					***
					IV
4.4.11	Prior to co COMPLETE Co	ommencing fuel ore Alteration	shuffle s Checklis	st 2.	second and the second second second second
	LOG the dat Alterations Control Log	ce and time th are started gbook.	at Core in the Uni	Lt	
4.4.12	During Core Alterations l hour, the alterations REPERFORM of required to specified to initiating	Alterations, s cease for gr en prior to co s, REFER to Ch those applicab be performed time frames pr core alterati	if Core eater thar mmencing c ecklist 2 le surveil within th ior to ons.	n core and llances ne	

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UNIT NO.	Construction of the state of th				INITIALS
4.5	POST-REFUE	LING OPERATIONS			
		CAUTION			
		Monitor Fuel Pool frequently to veri sealing.	level fy gate	is	
4.5.1	Upon compl post refue SEAL the F	etion of Core Alte ling verification, Fuel Pool To Transf	CLOSE a CLOSE a er Canal	and and L Gate.	
4.5.2	VERIFY tha in the sto LOCK the T	at the Fuel Transfe pred position, then Transfer Tube Gate	r Syster CLOSE a Valve.	n is and	
4.5.3	SHUT DOWN System per And Purifi	the Refueling Cavi 13719, "Spent Fue cation System".	ty Filtr 1 Coolir	ration ng	
4.5.4	TERMINATE PROCEED to (Mode 6 to	the use of this in 12000-C, "Refueli Mode 5)".	structiong Recov	on and very	-
COMPLETED					
00111 00100	Signa	ture	Date	Time	
REVIEWED:				1	
	Signa	iture	Date	Time	
COMMENTS:					
				State in the second second second second	
	A Reserved and the second s				

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5.0	REFERENCE	<u>s</u>	**
5.1	"Preparat Refuelin	ions For Refueling", ng Guidelines.	Westinghouse
5.2	PROCEDURE	S	
5.2.1	13011,	"Residual Heat Remov	al System"
5.2.2	13009,	"CVCS Reactor Makeup	Control System"
5.2.3	13105,	"Safety Injection Sy	stem"
5.2.4	13005,	"Reactor Coolant Sys	tem Draining"
5.2.5	13615,	"Condensate And Feed	water Systems"
5.2.6	13719,	"Spent Fuel Pool Coo System"	ling And Purification
5.2.7	14210,	"Containment Buildin Verification - Refu	g Penetrations eling"
5.2.8	12060-C,	"Refueling Recovery	(Mode 6 to Mode 5)"
5.2.9	93240-C,	"Reactor Vessel Assen Instructions"	mbly/Disassembly

END OF PROCEDURE TEXT

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	ŀ	10DE 6 ENTRY CHE	CKLIST 1 -		
UNIT NO.				IN	ITIALS
1,0	Prior to er following h during the requirement into Mode 6	ntering in Mode has been success specified inter ts therein are m	6, VERIFY fully comp val and the et for entr	the leted y	
1.1	Within 31 d	lays prior to en	tering Mode	6 :	
	a. 14228, Survei	"Operations Mo 1lance Logs".	nthly		
		Date	/ Time		
	b. 14514- Post-A Operab with i	C, "Fuel Handli ccident Exhaust ility Test". ( rradiated fuel	ng Building System Only applic in the FHB.	able	
			1		
		Date	Time		
1,2	Within 7 da	ys prior to ent	ering Mode	6:	
	a. 14225, Survei	"Operations We llance Logs",	ekly		
			1		
		Date	Time		
	b. 14423, Channe	Date "Source Range 1 1 Operational To	Time NIS Analog est".		
	b. 14423, Channe	Date "Source Range 1 1 Operational To Date	Time NIS Analog est". / Time	5 Automation 7 University	
1.3	b. 14423, Channe Within 72 h	Date "Source Range 1 1 Operational To Date ours prior to en	Time NIS Analog est". / Time ntering Mod	le 6:	
1.3	<ul> <li>b. 14423, Channe</li> <li>Within 72 h</li> <li>ENSURE that following r</li> </ul>	Date "Source Range 1 Operational To Date tours prior to en the more restrive activity condi-	Time NIS Analog est". / Time ntering Mod ictive of t tions is me	le 6:	
1.3	<ul> <li>b. 14423, Channe</li> <li>Within 72 h</li> <li>ENSURE that following r</li> <li>a. 14005, and DE necess</li> </ul>	Date "Source Range 1 Date Date ours prior to en the more restricativity condi "Shutdown Marg TERMINE that bo ary for Keff of	Time NIS Analog est". / Time ntering Mod ictive of t tions is me in Calculat ron concent less than	le 6: the t: ions", ration 0.95.	

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UNIT NO.	and the ball of the desired of the second			- INITIALS
	b. An RC Depar is eq	S boron analys tment and VER ual to or grea	sis from Ch LFY concent ater than 2	emistry ration 000 ppm.
	RCS B	oron	ppm	이 같은 것을 같은 것
			/	
		Date	e Time	
1.4	Within 12	hours prior to	entering 1	Mode 6:
	COMPLETE 1 Daily Logs therein ar	4000, "Operati ", and VERIFY e met for entr	ons Shift ) the require y into Mode	And sments e 6.
		1943년 - <u>1</u> 44	/	
		Date	Time	
REVIEWED:	Signature		Date Time	<u>.</u>
COMMENTS.			Succ alm	이 있었는 것 같은 것이 같은 것이 같이 많이 많이 했다.
CONFIGNES :				The second se
				And the second
				and a second
	Contract and the first sector of the first sector of the first sector of			contract of the state
			TRACT OF TRACTOR OF TRACTOR OF TRACT	
			an and an and a state of the second	
		nin med mennen state av de service de la	THE FACE STATES AND A SAME AND A SAME AND A SAME AND A	
		ANY ADDRESS OF THE OWNER AND A LOCATED AND A DESCRIPTION OF THE ADDRESS OF THE ADDRES		The structure of the st
		and the property of the second data of the second		ana tang ata ang ata pang ata ang ang ang ang ang ang ang ang ang an

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	cc	DRÉ ALTERATIONS CHECKLIS	Sheet 1 of 3 T 2
UNIT NO.			INITIALS
1.0	Prior to e VERIFY the	establishing Core Altera following:	tions,
1.1	Within 7 d Alteration Machine lo test has b (Only appl	lays prior to establishins, VERIFY the Fuel Hand bad test and crane inter been successfully comple licable prior to crane u	ng Core ling lock/scop ted. se.)
		Date / Tim	e
1.2	Within 100 Alteration	hours prior to establi as VERIFY the following:	shing Core
	a. The R been appli assem assem or co the r	efueling Machine Load T successfully completed. cable during movement o blies, rod control clus blies, thimble plug ass ntrol rod drive shafts eactor vessel.)	est has (Only f fuel ter emblies, within
		Date / Time	e
	b. The L and a movem React compl movem contr plug shaft	oad Test on each Auxilians sociated Load Indicator ent of Drive Rods within or Vessel has been succe eted. (Only applicable ent of fuel assemblies, ol cluster assemblies, assemblies, or control s within the reactor ver	ary Hoist r used for n the essfully during rod thimble rod drive ssel.)
		Date /	9
	c. NOTIF Conta Syste	Y Maintenance to reset inment Personnel Lock In m.	the nterlock
	d. VERIF Build Refue compl	Y that 14210, "Containme ing Penetrations Verific ling", has been satisfac eted.	ent cation- ctorily
		Data /	

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			Sheet 2 of 3
UNIT NO.			- INITIALS
1.3	Within 24 Core Alter	hours prior to establish ations:	ing
	NOTIFY Che required p During Ref	mistry to perform analys er 35180-C, "Chemistry C ueling".	is ontrol
		Date /	*
		Date lime	
1.4	Within 8 h Alteration	ours prior to establishi s:	ng Core
	COMPLETE 1 Channel Op	4423, "Source Range NIS erational Test".	Analog
		Date /	*
		Date IIme	
1.5	Within 2 h Alteration	ours prior to establishi s:	ng Core
	VERIFY tha is at leas (23 feet a (Only appl	t the Refueling Cavity w t 217 feet - 0 inches el bove the Reactor Vessel icable during fuel movem	ater level evation flange). ent.)
	Refueling	Cavity Level ft.	
		/	
		Date Time	
1.6	Within 1 h Alteration	our prior to establishin s:	g Core
	a. VERIF Contr appli estab Shift	Y communications between of Room and personnel at cable Refueling Stations lished using 14000, "Ope And Daily Logs".	the the has been rations
		/	
		Date Time	

the second se				PA	GENO.		
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UNIT NO.						INITI	ALS
	b. VERIFY	that the R	eactor has	been			
	subcri	tical for a	t least 100	hours	by		
	subcri	ticality.	e and time (Only appli	cable			
	during	movement o	f irradiate	d fuel	in		
	- 1	accor vebbe	***				
	Suberi	Date	Time				
	Comple	tod					
	compre	Date	Time				
REVIEWED:	87 an a france		1				
	Signature		Date Ti	me			
COMMENTS:							
	second local real second real printers of an or other second real						
	anna a sha a sh						