

September 11, 1995

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Mr. John F. Opeka
Executive Vice President - Nuclear
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, CT 06141-0270

SUBJECT: MILLSTONE 3 MOTOR-OPERATED VALVE INSPECTION 95-17

Dear Mr. Opeka:

This letter refers to the safety inspection conducted by Mr. D. Dempsey and others from June 19 - 30, 1995, at Millstone Nuclear Power Station, Unit 3. The inspectors focused on areas important to public health and safety, specifically, your program implemented in response to NRC Generic Letter (GL) 89-10, "Safety Related Motor-Operated Valve Testing and Surveillance." The preliminary findings were discussed with Mr. E. DeBarba and other members of your staff at the conclusion of the inspection.

The Millstone Unit 3 motor-operated valve (MOV) program was well-managed, and your initiative in enlisting the support of outside engineering expertise in anticipation of delayed completion of the industry's MOV Performance Prediction Program was noteworthy. We considered the corrective actions related to industry information concerning hydraulic locking of MOV actuator spring packs and with replacement of control switches that may contain Melamine parts to have been untimely and not coordinated well. In addition, the use of stall torque derived from generic motor curves when estimating valve actuator capability is contrary to vendor guidelines. Further, the technical justifications for program assumptions pertaining to valve factors, stem friction coefficients, load sensitive behavior, and lubrication degradation were not completed as of the end of this inspection. The NRC will review your response to these issues and additional information on MOV performance as part of future staff activities associated with Millstone 3 GL 89-10 program closure.

We also reviewed a design change intended to assure safety-grade cold shutdown capability in the event that two residual heat removal system MOVs became pressure locked. The strategy to perform a containment entry, for compensatory measures following certain design-basis events, was considered to be unacceptable. We understand that you subsequently concluded that these contingency actions no longer were required, and that you intend soon to restore the valves and system operating procedures to their original configuration. If your understanding of this is different, please contact us as soon as possible.

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No response to this letter is required. Your cooperation in these matters is appreciated, and we would be pleased to discuss our conclusions with you.

Sincerely,

Eugene M. Kelly, Chief
Systems Section
Division of Reactor Safety

Docket No. 50-423

Enclosure: Inspection Report No. 50-423/95-17

cc w/encl:

M. H. Brothers, Nuclear Unit Director
L. M. Cuoco, Esquire
F. R. Dacimo, Vice President, Haddam Neck Station
R. M. Kacich, Director, Nuclear Planning, Licensing and Budgeting
J. J. LaPlatney, Haddam Neck Unit Director
D. B. Miller, Senior Vice President, Millstone Station
Nicholas Reynolds, Esquire
S. E. Scace, Vice President, Nuclear Operations Services
J. F. Smith, Manager, Operator Training
J. M. Solymossy, Director, Nuclear Quality and Assessment Services
State of Connecticut SLO

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