

ATTACHMENT 1

REVISED PROPOSED CATAWBA TECHNICAL SPECIFICATIONS CHANGES

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.4.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE at least once per 18 months by:

- a. Performance of a CHANNEL CALIBRATION, and
- b. Operating the valve through one complete cycle of full travel.*

4.4.4.2 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel unless the block valve is closed with power removed in order to meet the requirements of ACTION b. or c. in Specification 3.4.4.

4.4.4.3 The ^{safety-related nitrogen} ~~emergency power~~ supply for the PORV ~~and block valves~~ shall be demonstrated OPERABLE at least once per 18 months by:

- a. Manually transferring motive power from the normal (air) supply to the emergency (nitrogen) supply,
- b. Isolating and venting the normal (air) supply, and
- c. Operating the valves through a complete cycle of full travel.

* In order to simulate environmental effects representative of operating conditions, SR 4.4.4.1b. should be conducted when the Reactor Coolant System temperature is greater than 200°F; however, this SR shall not be performed in Modes 1 or 2.

CATAWBA - UNITS 1 & 2

3/4 4-11

Amendment No. ~~V~~(Unit 1)
Amendment No. ~~A~~(Unit 2)

~~9/16/86~~

REACTOR COOLANT SYSTEM

BASES

SAFETY VALVES (Cont.)

relief capability and will prevent overpressurization. In addition, the Overpressure Protection System provides a diverse means of protection against overpressurization at low temperatures.

During operation, all pressurizer Code safety valves must be OPERABLE to prevent the Reactor Coolant System from being pressurized above its Safety Limit of 2735 psig. The combined relief capacity of all of these valves is greater than the maximum surge rate resulting from a complete loss-of-load assuming no Reactor trip until the first Reactor Trip System Trip Setpoint is reached (i.e., no credit is taken for a direct Reactor trip on the loss-of-load) and also assuming no operation of the power-operated relief valves or steam dump valves.

Demonstration of the safety valves' lift settings will occur only during shutdown and will be performed in accordance with the provisions of Section XI of the ASME Boiler and Pressure Code.

3/4.4.3 PRESSURIZER

The limit on the maximum water volume in the pressurizer assures that the parameter is maintained within the normal steady-state envelope of operation assumed in the SAR. The limit is consistent with the initial SAR assumptions. The 12-hour periodic surveillance is sufficient to ensure that the parameter is restored to within its limit following expected transient operation. The maximum water volume also ensures that a steam bubble is formed and thus the Reactor Coolant System is not a hydraulically solid system. The requirement that a minimum number of pressurizer heaters be OPERABLE enhances the capability of the plant to control Reactor Coolant System pressure and establish natural circulation.

3/4.4.4 RELIEF VALVES

The power-operated relief valves (PORVs) and steam bubble function to relieve Reactor Coolant System pressure during all design transients up to and including the design step load decrease with steam dump. ~~Operation of the PORVs minimizes the undesirable opening of the spring-loaded pressurizer Code safety valves.~~ Each PORV has a remotely operated block valve to provide a positive shutoff capability should a relief valve become inoperable. Testing of the PORVs includes the emergency N₂ supply from the Cold Leg Accumulators. INSERT A This test demonstrates that the valves in the supply line operate satisfactorily and that the nonsafety portion of the instrument air system is not necessary for proper PORV operation.

3/4.4.5 STEAM GENERATORS

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the Reactor Coolant System will be maintained. The program for inservice inspection of steam

INSERT FOR TECHNICAL SPECIFICATIONS

INSERT A (for Bases Section 3/4.4.4)

The OPERABILITY of the PORVs and block valves is determined on the basis of their being capable of performing the following functions: 1) Manual control of PORVs to control Reactor Coolant System pressure. This is a function that is used for the steam generator tube rupture accident coincident with a loss of all offsite power and for plant shutdown. 2) Maintaining the integrity of the reactor coolant pressure boundary. This is a function that is related to controlling identified leakage and ensuring the ability to detect unidentified reactor coolant pressure boundary leakage. 3) Manual control of the block valve to unblock an isolated PORV to allow it to be used for manual control of Reactor Coolant System pressure and isolate a PORV with excessive seat leakage. 4) Automatic control of PORVs to control Reactor Coolant System pressure except for limited periods where the PORV has been isolated due to excessive seat leakage and except for limited periods where the PORV and/or block valve is closed because of testing and is fully capable of being returned to its normal alignment at any time, provided that this evolution is covered by an approved procedure. This is a function that reduces challenges to the code safety valves for overpressurization events. 5) Manual control of a block valve to isolate a stuck-open PORV.