

NOTICE OF VIOLATION

Commonwealth Edison
Braidwood Nuclear Power Station

Docket No. NPF-72
License No. 50-456

During an NRC inspection conducted on September 19 through December 17, 1991, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1991), the violation is listed below:

10 CFR Part 50, Appendix J, Section III.A.1.a requires, in part, that, if during a Type A test potentially excessive leakage paths are identified which will interfere with satisfactory completion of the test, the Type A test shall be terminated and the leakage through such paths shall be measured using local leakage testing methods. The corrective action taken and the change in leakage rate determined from the tests and the overall integrated leakage determined from the local leak and Type A tests shall be included in the report submitted to the Commission.

Contrary to the above, on February 12 through 14, 1991, during performance of the Unit 1 Type A test, potentially excessive leakage paths were identified which interfered with satisfactory completion of the test and the Type A test was not terminated nor was the leakage through such paths measured.

This is a Severity Level IV violation. (Supplement 1)

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555, with a copy to the U.S. Nuclear Regulatory Commission, Region III, 799 Roosevelt Road, Glen Ellyn, Illinois, 60137, and a copy to the NRC Resident Inspector at the Braidwood site, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violation, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
This 31 day of January, 1992



H. J. Miller, Director
Division of Reactor Safety