

## NUCLEAR REGULATORY COMMISSION

DOCKET TICE

WASHINGTON, D.C. 20585-0001

September 11, 1995

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 OPUS Place, Suite 500
Downers Grove, IL 60515

SUBJECT: SUPPLEMENTAL SAFETY EVALUATION REGARDING THE PROPOSED CORE SHROUD

REPAIR - QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 (TAC NOS.

M91301 AND M91302)

Dear Mr. Farrar:

In teleconferences held June 14 and June 16, 1995, Commonwealth Edison Company (ComEd) informed the staff of an event which caused inadvertent loading on the core shroud repair hardware at Quad Cities Nuclear Station, Unit 2.

By letter dated June 19, 1995, ComEd submitted a revised 10 CFR 50.59 safety evaluation of the core shroud repair that addresses the inadvertent loading of two of the tie rod stabilizer assemblies. This loading event occurred as a result of interference of the shroud head/steam separator assembly support legs and the long upper supports of two tie rod stabilizer assemblies during reassembly of the reactor vessel internals. ComEd concluded that the safety of the long upper supports had not been compromised as a result of this event. On June 30 and July 6, 1995, ComEd submitted additional justification to support their conclusion.

The staff has reviewed the effects on the repair hardware due to the inadvertent loading, and concludes that the ability of the repair hardware to perform its intended safety function does not appear to have been adversely affected as a result of this inadvertent loading event.

In accordance with 10 CFR 50.59, ComEd also determined that no unreviewed safety question had been introduced as a result of the event and that no Technical Specification revision is involved. The staff agrees with this determination and concludes that no license amendment, pursuant to 10 CFR 50.90, is necessary.

The original Safety Evaluation (SE) issued by the NRC on June 8, 1995, contained inaccuracies. Correction of these inaccuracies are addressed in the enclosed supplemental SE. However, the conclusion of the original SE is not affected by these changes.

In addition to the items addressed in this letter and supplemental SE, the licensee informed the staff on August 28, 1995, that while preparing a

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response to an NRC Request for Additional Information (RAI) related to the proposed Dresden, Units 2 and 3, core shroud repair, ComEd identified a discrepancy in the original General Electric (GE) calculations for the seismic analysis of the reactor internals. The discrepancy affected the original Quad Cities, Units 1 and 2, seismic analysis of the reactor internals and was present during the preparation of the core shroud repair seismic analysis for these units, since it was not yet identified. The discrepancy related to the underestimation of the total mass at one reactor internals node point in the seismic model. In a teleconference on August 31, 1995, the licensee informed the staff of the circumstances that led to the identification of the discrepancy and of the preliminary results of its assessment of the effect of the discrepancy on the adequacy of the core shroud repair just completed at Quad Cities, Unit 2. The licensee submitted this preliminary assessment result in a letter to the staff on September 5, 1995. Based on its evaluation of the licensee's submittal, the staff concluded that the identification of the calculational error does not alter its initial conclusion regarding the adequacy of the core shroud repair. The staff's conclusion regarding the operability of Quad Cities, Unit 1 (without the tie rods installed), as stated in SE's dated July 21, 1994, and August 16, 1995, are also unaffected by this error. The licensee indicated that formal correspondence will be transmitted to the staff upon completion of its final evaluation of the discrepancy's impact on Quad Cities, Unit 2, in mid-October 1995. The staff will issue a new supplement to the SE upon completion of its evaluation of the submittal.

Our evaluation of the issues addressed in this correspondence is provided in the enclosed supplemental SE. Also enclosed please find the revised original SE.

Sincerely,

Original signed by
Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-254, 50-265

Enclosures: 1. Supplemental Safety Evaluation

2. Revised Safety Evaluation

cc w/encls: see next page

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J. Roe (JWR) R. Capra G. Carpenter, 014B2 C. Moore
R. Pulsifer OGC, 015B18 ACRS (4), T2E26 P. Hiland, I

R. Pulsifer OGC, 015B18 ACRS (4), T2E26 P. Hiland, RIII R. Wessman, 07E23 M. Hartzman, 07E23 J. Strosnider, 07D4 R. Jones, 08E23

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D. L. Farrar Commonwealth Edison Company

cc:

Mr. Stephen E. Shelton Vice President Iowa-Illinois Gas and Electric Company P.O. Box 4350 Davenport, Iowa 52808

Michael I. Miller, Esquire Sidley and Austin One First National Plaza Chicago, Illinois 60603

Mr. L. William Pearce Station Manager Quad Cities Nuclear Power Station 22710 206th Avenue North Cordova, Illinois 61242

U.S. Nuclear Regulatory Commission Quad Cities Resident Inspectors Office 22712 206th Avenue North Cordova, Illinois 61242

Chairman
Rock Island County Board
of Supervisors
1504 3rd Avenue
Rock Island County Office Bldg.
Rock Island, Illinois 61201

Illinois Department of Nuclear Safety Office of Nuclear Facility Safety 1035 Outer Park Drive Springfield, Illinois 62704

Regional Administrator U.S. NRC, Region III 801 Warrenville Road Lisle, Illinois 60532-4351

Richard J. Singer Manager - Nuclear MidAmerican Energy Company 907 Walnut Street P.O. Box 657 Des Moines, Iowa 50303 Quad Cities Nuclear Power Station Unit Nos. 1 and 2

Maria America

Brent E. Gale, Esq.
Vice President - Law and
Regulatory Affairs
One RiverCenter Place
106 East Second Street
P.O. Box 4350
Davenport, Iowa 52808

Warren Bilanin, EPRI Task Manager 3412 Hillview Ave. Palo Alto, California 94303

Robin Dyle, Technical Chairman BWRVIP Assessment Task Southern Muclear Operating Co. P.O. Box 236 40 Inverness Center Parkway Birmingham, Alabama 35201