



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30303

Report Nos.: 50-424/84-08 and 50-425/84-08

Licensee: Georgia Power Company
P. O. Box 4545
Atlanta, GA 30302

Docket Nos.: 50-424 and 50-425

License Nos.: CPPR-108 and CPPR-109

Facility Name: Vogtle 1 and 2

Inspection at Vogtle site near Waynesboro, Georgia

Inspector: John F. Sanders
W. F. Sanders

5/15/84
Date Signed

Approved by: V. W. Panciera
V. W. Panciera, Section Chief,
Project Section 2B
Division of Reactor Projects

5/15/84
Date Signed

SUMMARY

Inspection on March 8 - April 12, 1984

Areas Inspected

This routine, unannounced inspection involved 192 resident inspector-hours on site in the areas of Diesel Generator Reconditioning, Primary Containment, Piping, Concrete Placement, Auxiliary and Control Building Activities, and Welder Qualification.

Results

Of the areas inspected, no violations or deviations were identified.

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REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *W. T. Nickerson, Deputy General Project Manager
- *H. H. Gregory III, General Manager, Vogtle Nuclear Construction Department
- *M. H. Googe, Assistant Project Construction Manager
- P. D. Rice, General Manager, Quality Assurance
- *C. W. Hayes, Vogtle Quality Assurance Manager
- *R. W. McManus, Manager of Quality Control
- *G. A. McCarley, Project Compliance Coordinator
- *J. R. Petro, Senior Quality Assurance Field Representative
- *C. Sarver, Jr., Senior Quality Assurance Engineer

Other licensee employees contacted included craftsmen, technicians, supervision, inspectors and office personnel.

Other Organizations

- *J. Ruud, Field Quality Assurance Supervisor, Bechtel Power Co.
- D. L. Kinnish, Project Field Engineer, Bechtel Power Co.
- *J. Mamon, Quality Engineer, Project Field Engineering, Bechtel Power Co.
- D. Wieland, Site Manager, Westinghouse Electric Corporation

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on April 12, 1984, with those persons indicated in paragraph 1 above. The Inspector described the areas inspected and discussed in detail the inspection findings.

3. Licensee Action on Previous Enforcement Matters

Not inspected.

4. Unresolved Items

Unresolved items are matters about which more information is required to determine whether they are acceptable or may involve violations or deviations. New unresolved items identified during this inspection are discussed in paragraph 6. One previous unresolved item is discussed in paragraph 7.

5. Construction Inspection

Periodic inspections were made throughout this reporting period in the form of general type inspections in different areas of both facilities. The areas were selected on the basis of the scheduled activities and were varied to provide wide coverage. Observations were made of activities in progress to note defective items or items of noncompliance with the required codes and regulatory requirements. On these inspections, particular note was made of the presence of quality control inspectors, supervisors, and quality control evidence in the form of available process sheets, drawings, material identification, material protection, performance of tests, and housekeeping.

Interviews were conducted with craft personnel, supervisors, coordinators, quality control inspectors, and others as they were available in the work areas.

Observations were made in the following areas:

- a. Ongoing work activities in progress in the Primary Containment for Units 1 and 2, Auxiliary Building, Control Building, Nuclear Service Cooling Water Structures, Diesel Generator Building, Concrete Placement, and Category 1 Backfill.

6. Reactor Pressure Vessel Documentation

A review of the hard copy documentation package for the Unit 1 Reactor Pressure Vessel (RPV) and a followup inspection of the microfiche data revealed several inconsistencies and concerns. The hard copy file contained the description and partial record for two additional longitudinal weld seams and major plate repairs which would be subject to inclusion in the ASME Section XI preservice and inservice program. This information was subsequently found to be applicable to the Unit 2 RPV. After checking the Unit 2 ASME Code Data report form (N-1 Manufacturer Data Report for Nuclear Vessels) it was noted that no mention of these repairs were noted on the N-1 form. This was discussed with the Georgia Power Company Sr. Quality Assurance Field Representative and the Westinghouse Site Manager. After discussions with other Westinghouse personnel, the Site Manager reported that there is no requirement for listing repairs or additional weld seams on the Code Data Report. This is inconsistent with the Code Data Report on Unit 1, Para 19 which describes a repair to weld seam 201-121A. The inspector expressed concern that the communication system, the document review system, does not appear to be structured to identify and incorporate repairs and modifications into the Section XI inservice inspection program. The General Manager of Construction stated that this problem would be reviewed. These are considered to be unresolved items 50-424 and 425/84-08-01.

7. R.H.R. Pump Nozzle Transitions

Unresolved Item 50-424 and 50-425/83-10-01 (Open): Two meetings were held during this reporting period to discuss and resolve the concerns related to the 45° machined grooves and the undersize radii in the bottom of the grooves in the inlet and outlet nozzles of the R.H.R. pumps. The first meeting was March 7, 1984, and was attended by following persons:

Clancy Benton	-	Westinghouse Project Engineer
Don Wieland	-	Westinghouse Site Manager
Ozen Batum	-	Georgia Power, General Manager, Project Engineering and Licensing
Ed Groover	-	Site Q.A. Manager
W. F. Sanders	-	NRC Senior Resident Inspector

The second meeting was held March 21, 1984, and was attended by:

Ulm McKerson	-	Deputy General Manager
Ozen Batum	-	General Manager, Project Engineering and Licensing
Ed Groover	-	Site Q.A. Manager
Don Wieland	-	Westinghouse Site Manager
Bill Reed	-	Westinghouse Field Engineer
Hugh Dance	-	Branch Chief, U.S.N.R.C.
W. F. Sanders	-	NRC Senior Resident Inspector

The original concerns were reiterated and discussed. Westinghouse stated that the geometry of grooves located on the suction and discharge nozzles of the pump could be justified based on measurements taken by Westinghouse which showed the radius at the bottom of the grooves to be 0.125" and 0.063" for the suction and discharge nozzles respectively. These radii exceed the minimum radius of 0.05 x nozzle thickness (0.05T min.) specified in paragraph NC-4250 of the ASME Code, Section III, 1980 Edition. However, the inspector noted that the RHR pump casing is required to be designed to the 1971 Edition, 1972 Summer Addenda of the code. Further the geometry of the discharge nozzle groove is not in accordance with WNES Drg No. 271C 9000 which specifies a 45° slope with a 0.12" minimum radius at the bottom of the groove.

The meeting was concluded with assurances that Georgia Power Engineering would evaluate the 45° transition and size of radii and provide justification addressed to service conditions, stress concentrations, etc. This item continues as unresolved and will remain open pending appropriate justification.

8. Diesel Generator Building Concrete Placement

An inspection was made on the concrete placement, 1-071-001C, for the Diesel Generator Building base slab (9' 0"). The inspection included the forms, embedments and reinforcing steel installation for proper placement, leak tightness and cleanliness. Observations were made of the crew and equipment during placement activities - vibrators used properly, chutes of proper

length, Quality Control inspections during placement, and proper curing procedure after placement. No violations or deviations were identified.

9. Emergency Diesel Generators

Periodic inspections were made throughout this reporting period to observe the ongoing activities related to the inspection and modification project of the Trans DeLaval Diesel Generators. A review was made of two controlling procedures:

- o Emergency Diesel Generator Maintenance, VEGP-2403-001. This procedure provides instruction for the disassembly, inspection and reassembly of the DeLaval RV-16-4 diesel engine.
- o Emergency Diesel Generator Inspection and Modification Project Quality Control, VEGP-2403-002. This procedure is to provide a method of prescribing and documenting the Quality Control measures to be followed for the diesel generator inspection and modification project.

The following activities were observed:

- a. Disassembly of Heads
- b. Removal and Visual Examination of Skirts
- c. Removal and Visual Examination of Pistons
- d. Removal and Visual Examination of Sleeves
- e. Removal and Visual Examination of Connecting Rods
- f. Fluorescent Penetrant Inspection of (2) heads.
- g. Liquid Penetrant Inspection of (2) Piston Rods and Bearings

No Violations or Deviations were observed.